

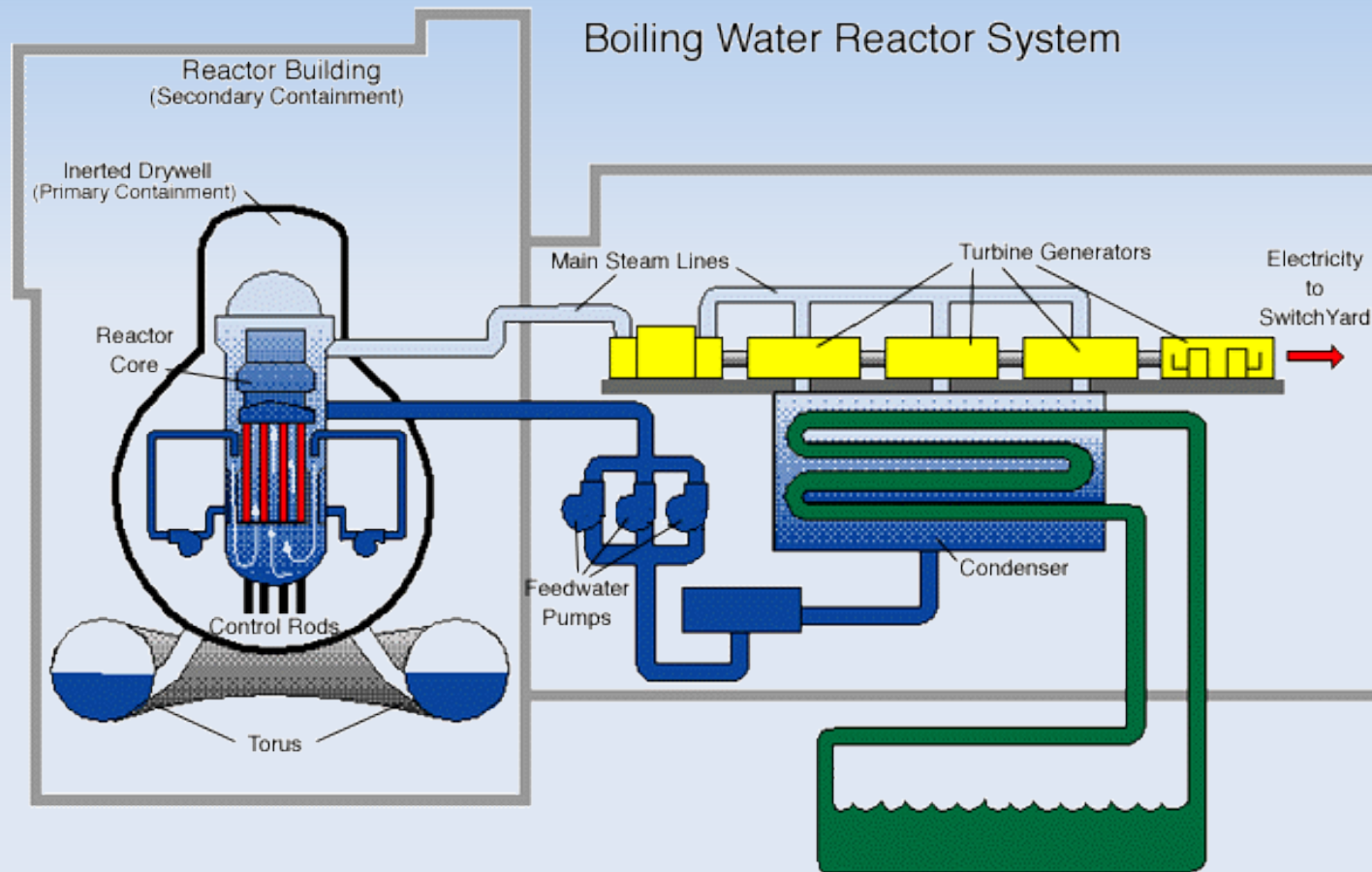
Fukushima, Chronologie eines Unfalls

- Struktur des Vortrags
 - Ablauf des Reaktorunfalls
 - Technische Maßnahmen für die Zukunft
 - Radiologie
 - An Land
 - Zu Wasser

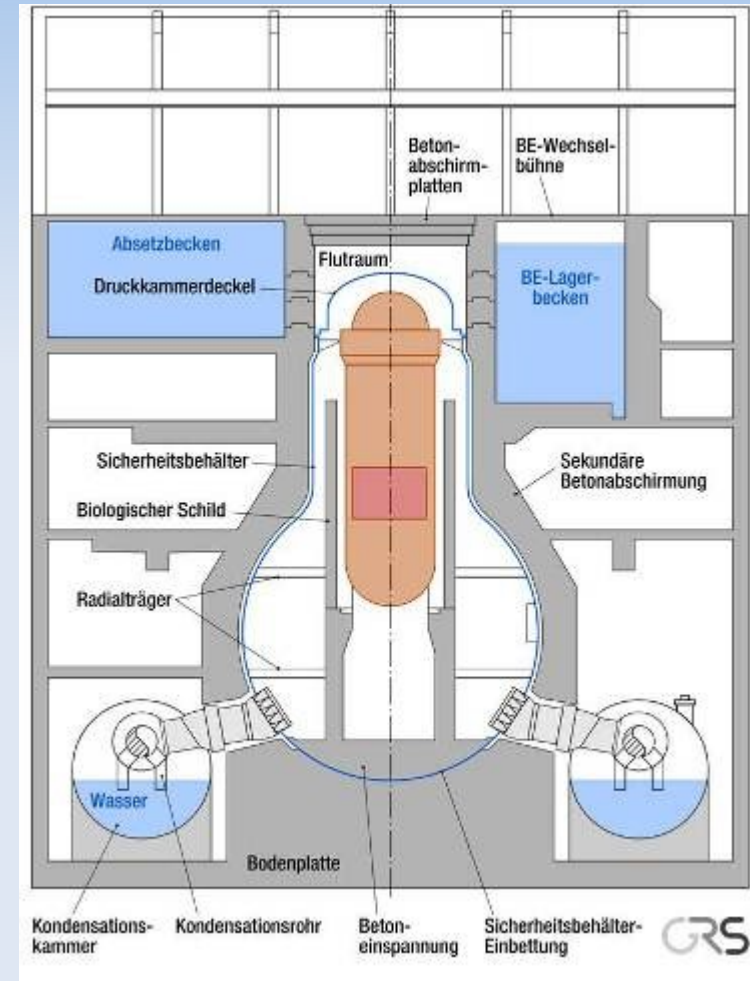
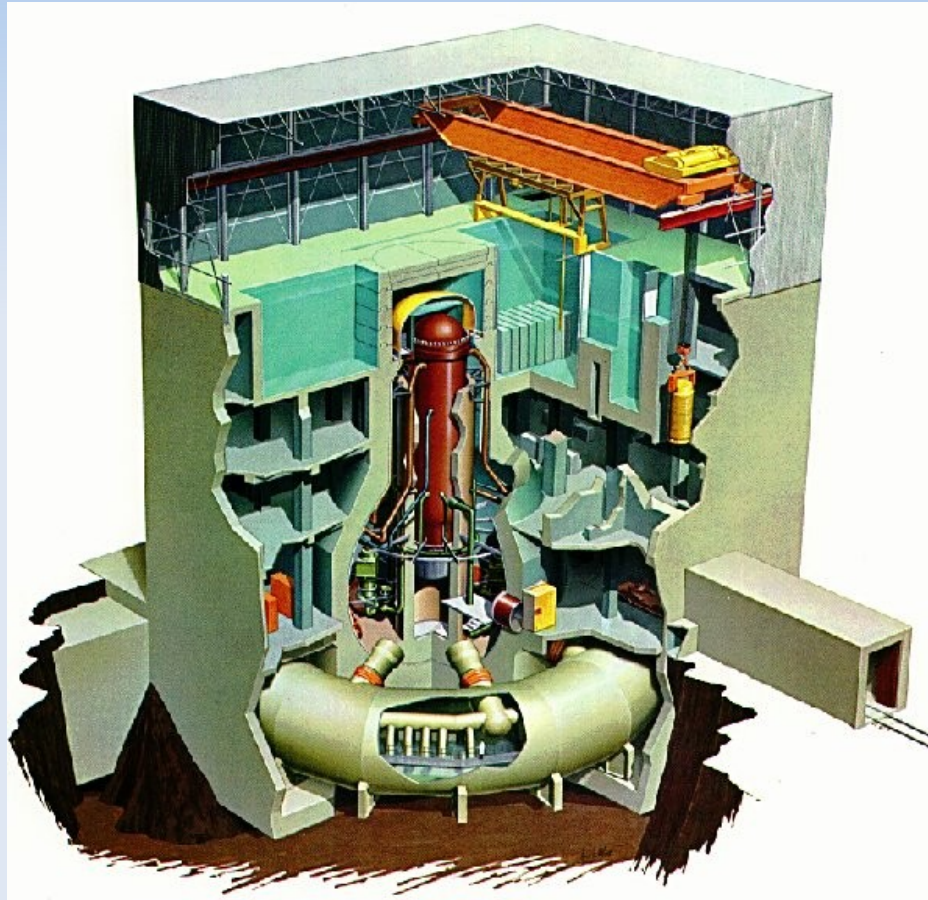
Reaktordesign



Reaktordesign



Reaktordesign



Reaktordesign

Reactor Service Floor
(Steel Construction)

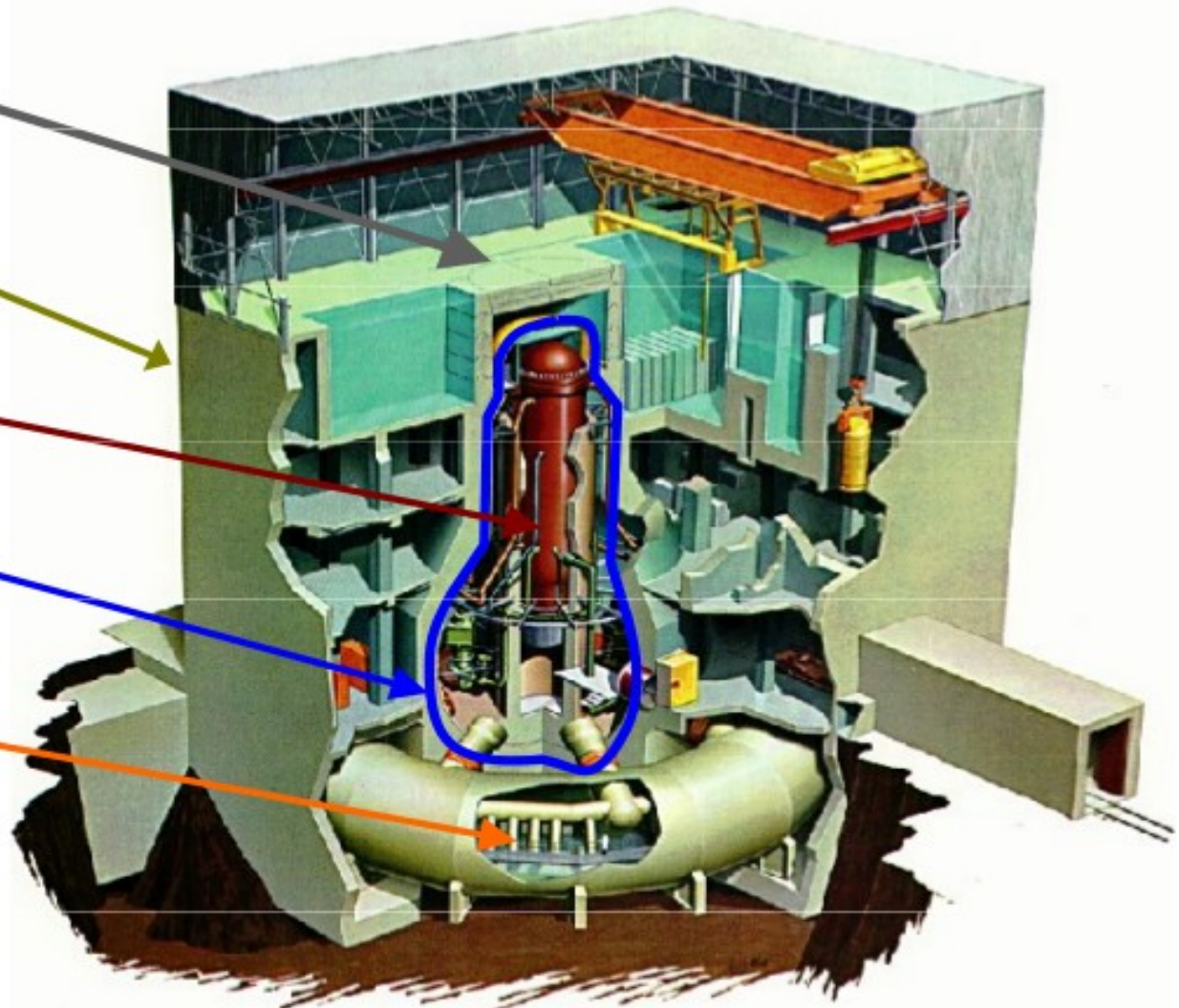
Concrete Reactor Building
(Secondary Containment)

Reactor Pressure Vessel

Primary Containment
(Drywell)

Pressure Suppression Pool
(Wetwell)

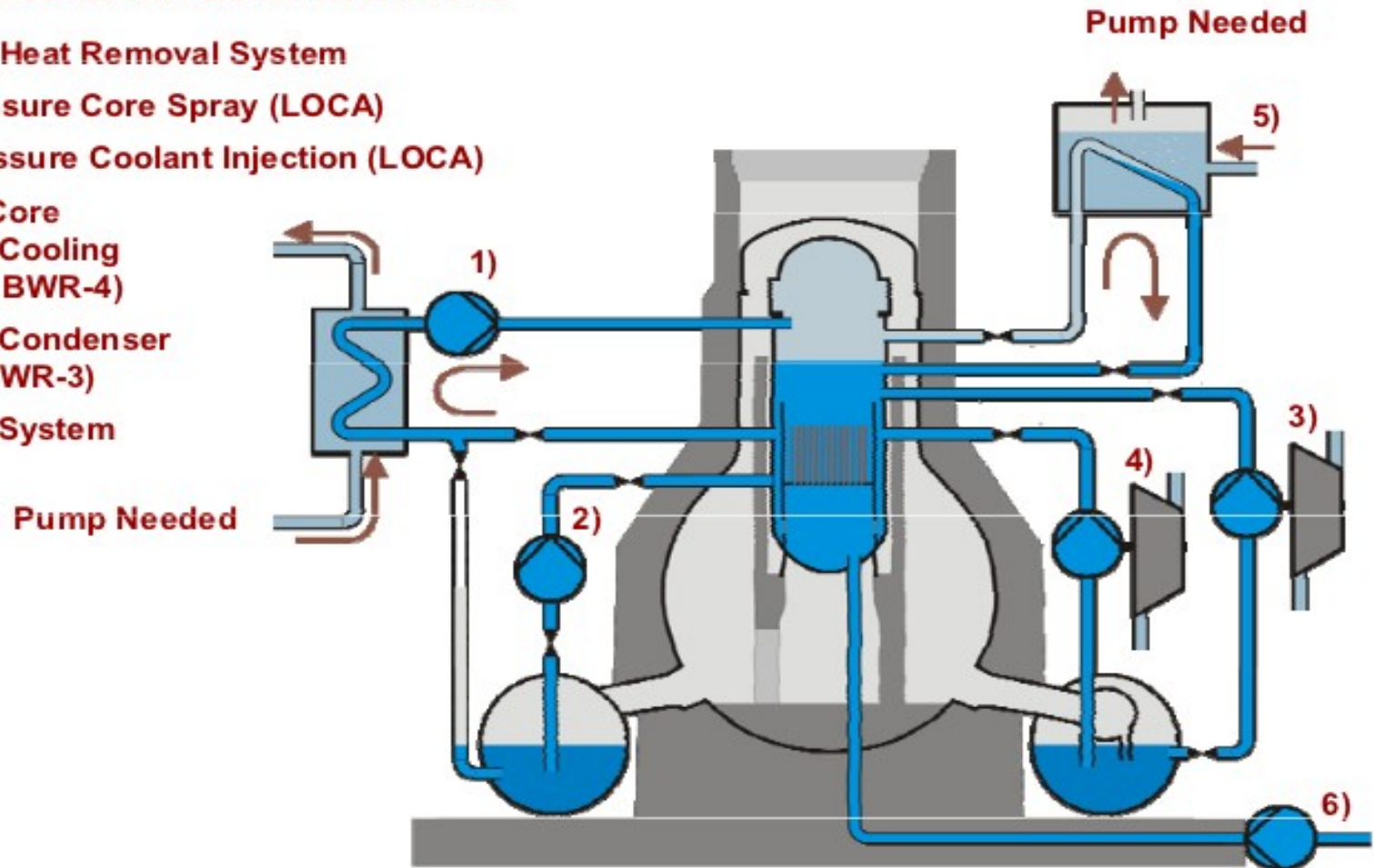
- ▶ Reactor: BWR-3
- ▶ Containment: Mark-I



Reaktordesign

Emergency Core Cooling Systems of Different Units at Fukushima Daiichi

- 1) Residual Heat Removal System
- 2) Low-Pressure Core Spray (LOCA)
- 3) High-Pressure Coolant Injection (LOCA)
- 4) Reactor Core Isolation Cooling (Unit 2/3: BWR-4)
- 5) Isolation Condenser (Unit 1: BWR-3)
- 6) Borating System



Reaktortypen

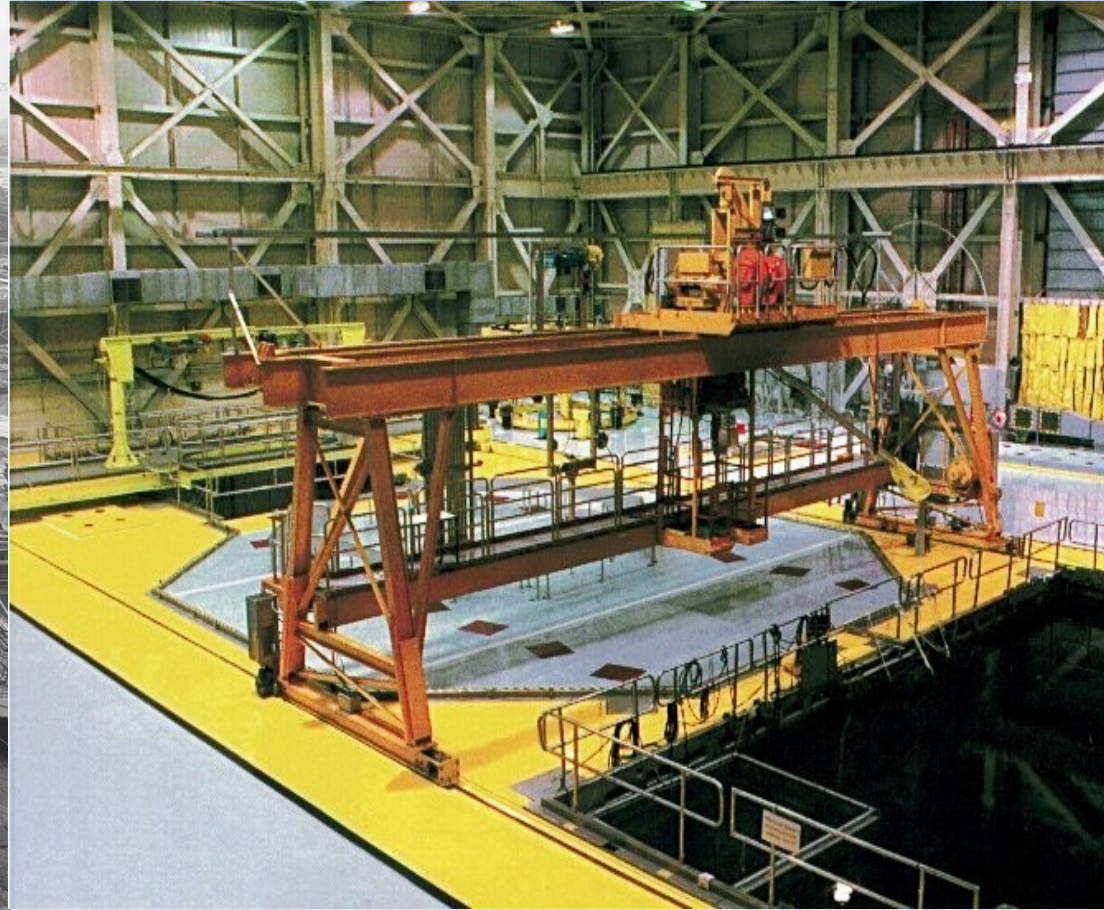
	Unit 1	Unit 2	Unit 3	Unit 4	Unit 5	Unit 6
	BWR-3	BWR-4	BWR-4	BWR-4	BWR-4	BWR-5
PCV Model	Mark-1	Mark-1	Mark-1	Mark-1	Mark-1	Mark-2
Electric Output (MWe)	460	784	784	784	784	1100
Max. pressure of RPV	8.24MPa	8.24MPa	8.24MPa	8.24MPa	8.62MPa	8.62MPa
Max. Temp of the RPV	300°C	300°C	300°C	300°C	302°C	302°C
Max. Pressure of the CV	0.43MPa	0.38MPa	0.38MPa	0.38MPa	0.38MPa	0.28MPa
Max. Temp of the CV	140°C	140°C	140°C	140°C	138°C	171°C(D/W) 105°C(S/C)
Commercial Operation	1971,3	1974,7	1976,3	1978,10	1978,4	1979,10
Emergency DG	2	2	2	2	2	3*
Electric Grid	275kV × 4				500kV × 2	
Plant Status on Mar. 11	In Operation	In Operation	In Operation	Refueling Outage	Refueling Outage	Refueling Outage

* One Emergency DG is Air-Cooled

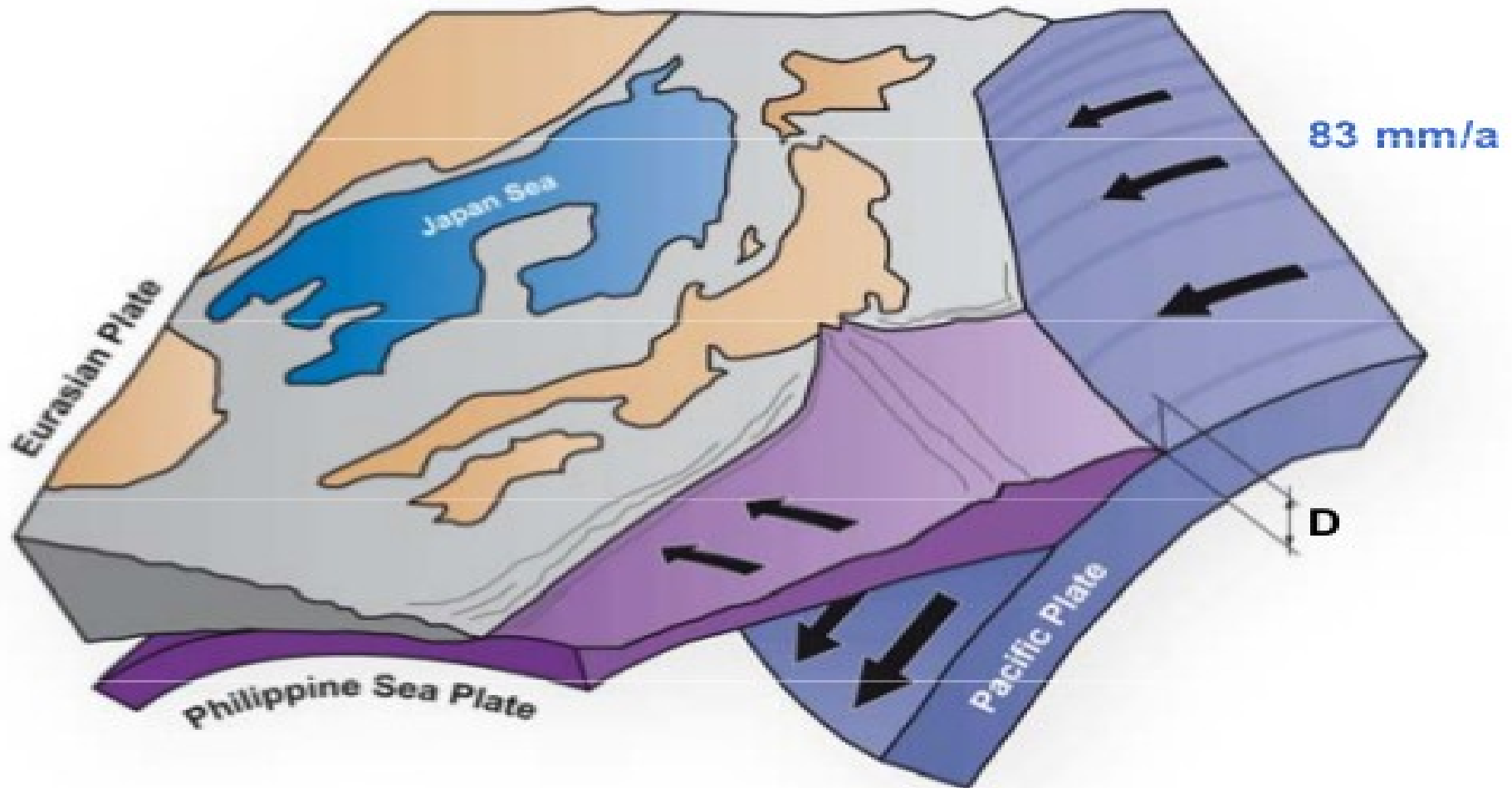
Reaktordesign

Unit	1	2	3	4	5	6
Number of Fuel Assembly in the Core	400	548	548	-	548	764
Number of Spent Fuel Assembly in the Spent Fuel Pool	292	587	514	1,331	946	876
Number of New Fuel Assembly in the Spent Fuel Pool	100	28	52	204	48	64
Water Volume (m ³)	1,020	1,425	1,425	1,425	1,425	1,497

Reaktordesign

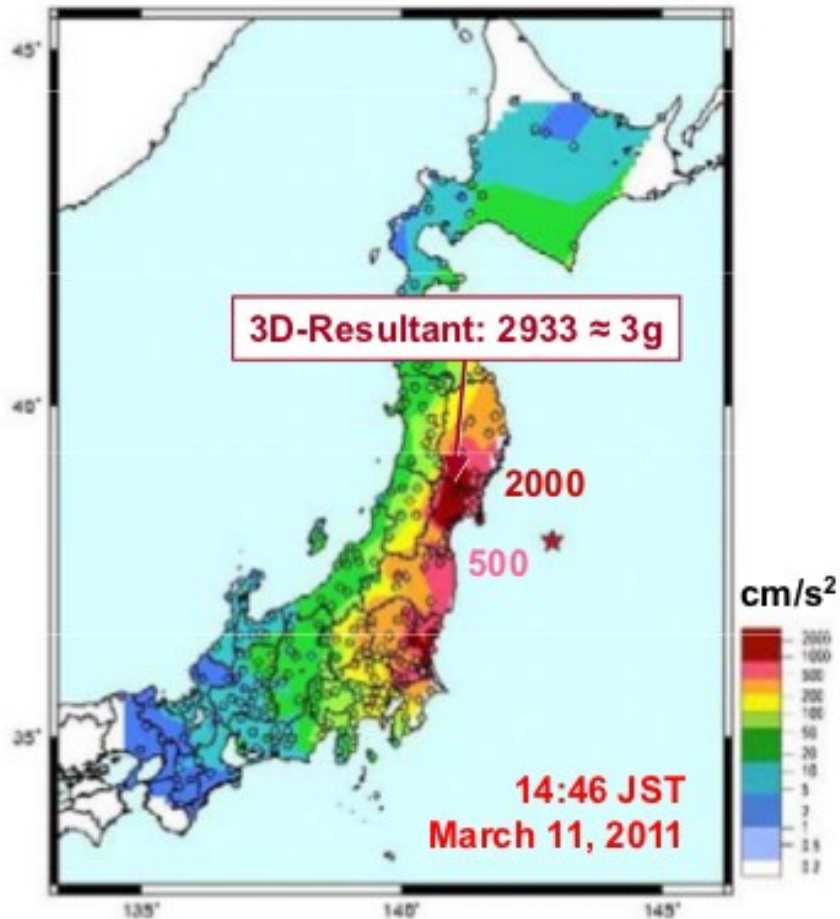


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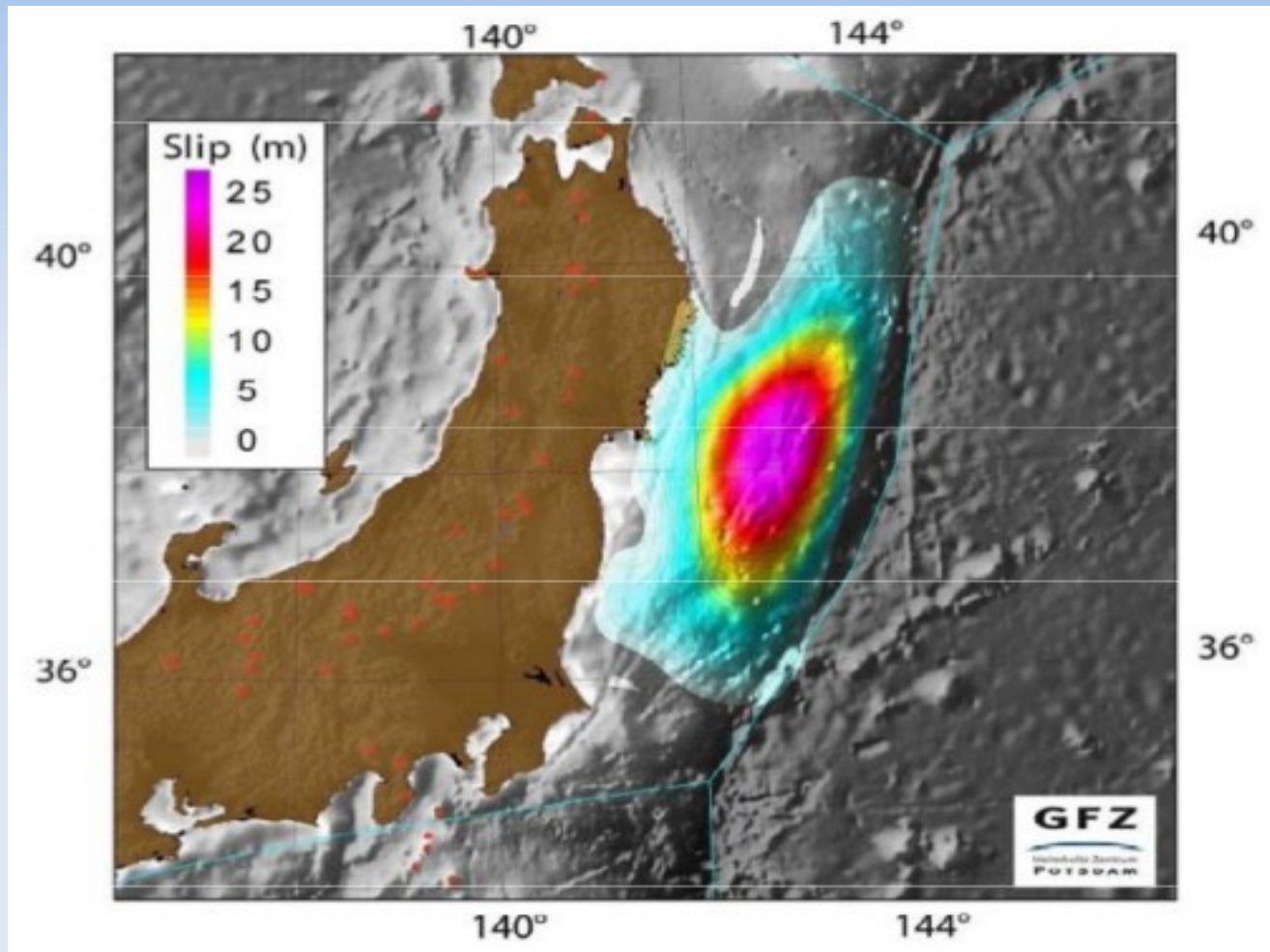
Chronologie eines Unfalls

Peak Accelerations Contour Map



Fukushima	Acceleration 1) in cm/s ²		
	Horizontal		Vertical
	N-S	E-W	
Daiichi-1	460	447	258
Daiichi-2	348	550	302
Daiichi-3	322	507	231
Daiichi-4	281	319	200
Daiichi-5	311	548	256
Daiichi-6	298	444	244
Design Basis	441	438	412
Daini-1	254	230	305
Daini-2	243	196	232
Daini-3	277	216	208
Daini-4	210	205	288
Design Basis	415	415	504
Shutdown 2)	135 to 150		100

Chronologie eines Unfalls



Unit	1	2	3	4
Power (MWe /MWth)	460/1380	784/2381	784/2381	784/2381
Type of Reactor	BWR-3	BWR-4	BWR-4	BWR-4
Status at time of EQ	In service – auto shutdown	In service – auto shutdown	In service – auto shutdown	Outage

Chronologie eines Unfalls



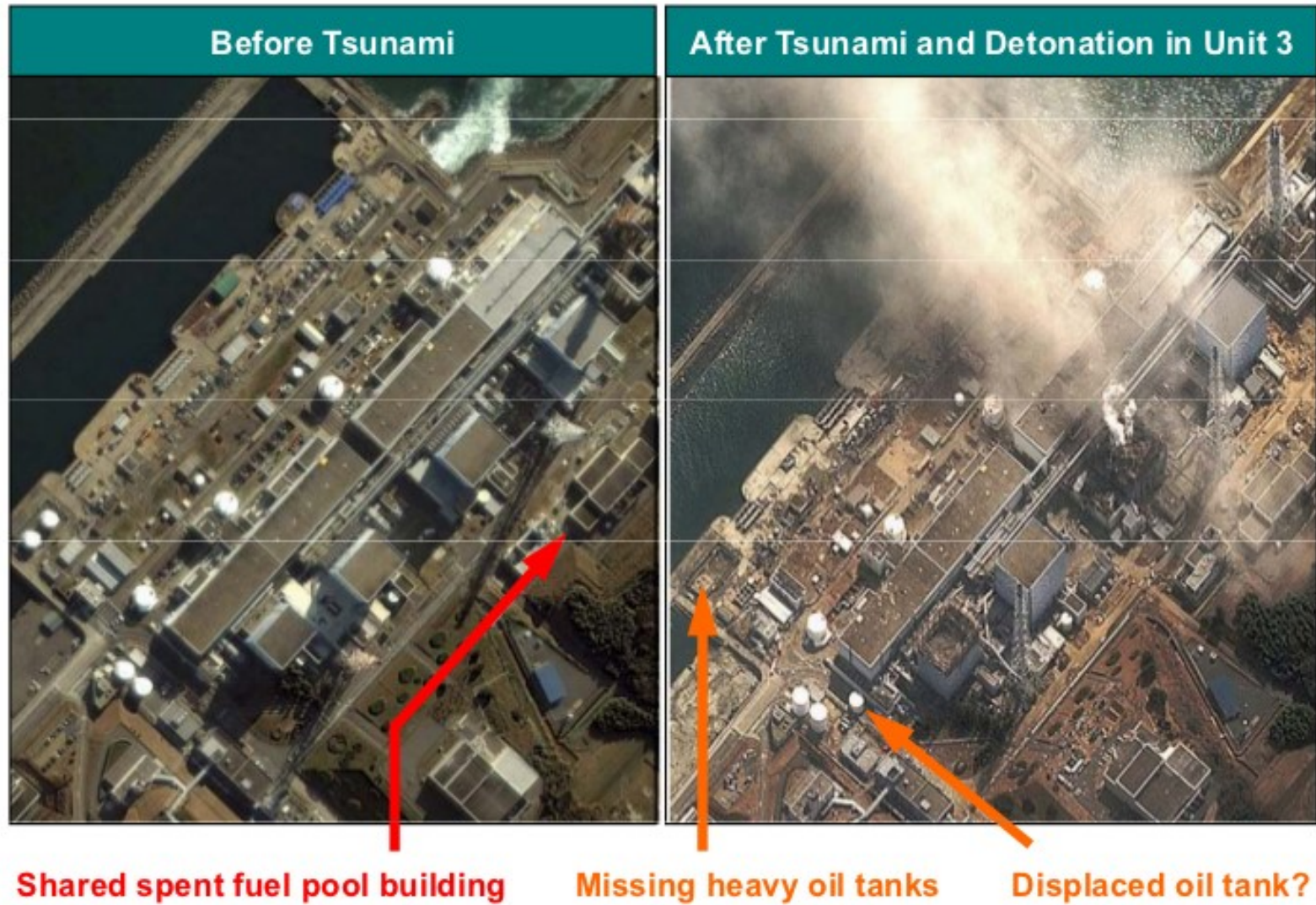
Chronologie eines Unfalls



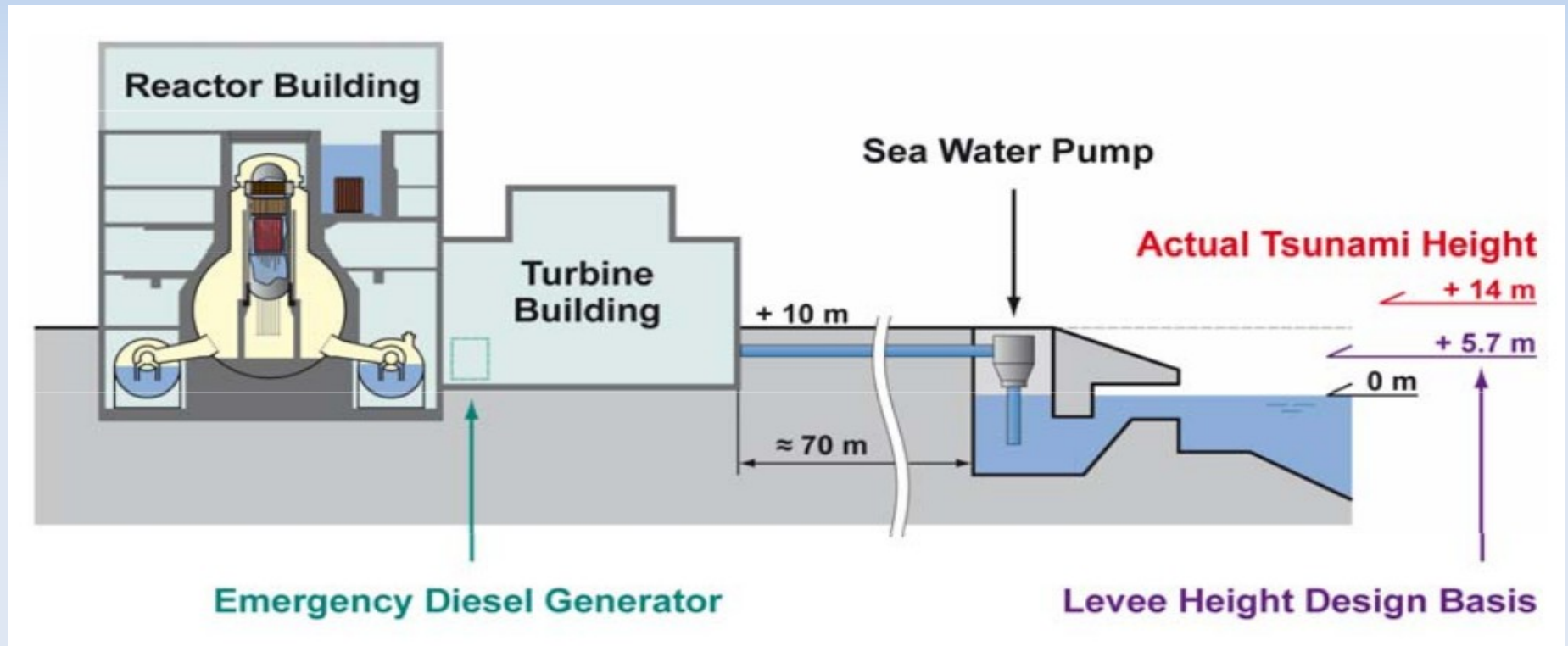
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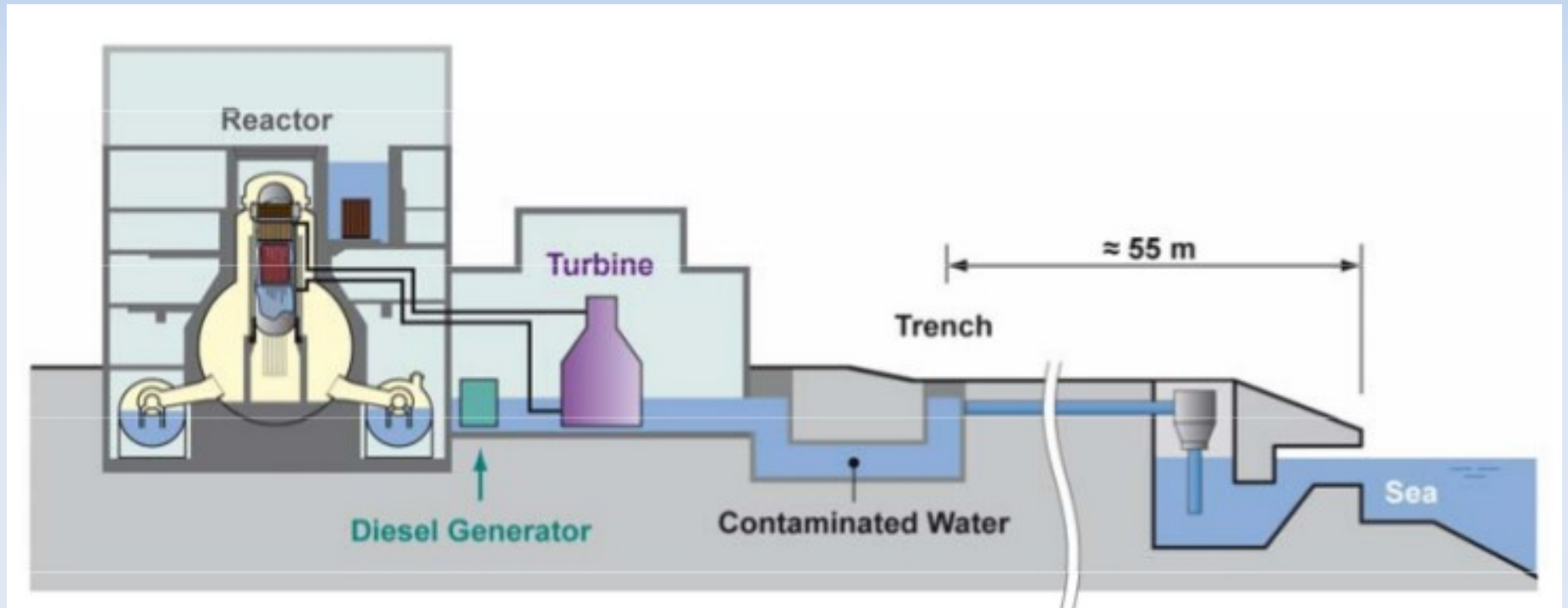
Chronologie eines Unfalls



Chronologie eines Unfalls

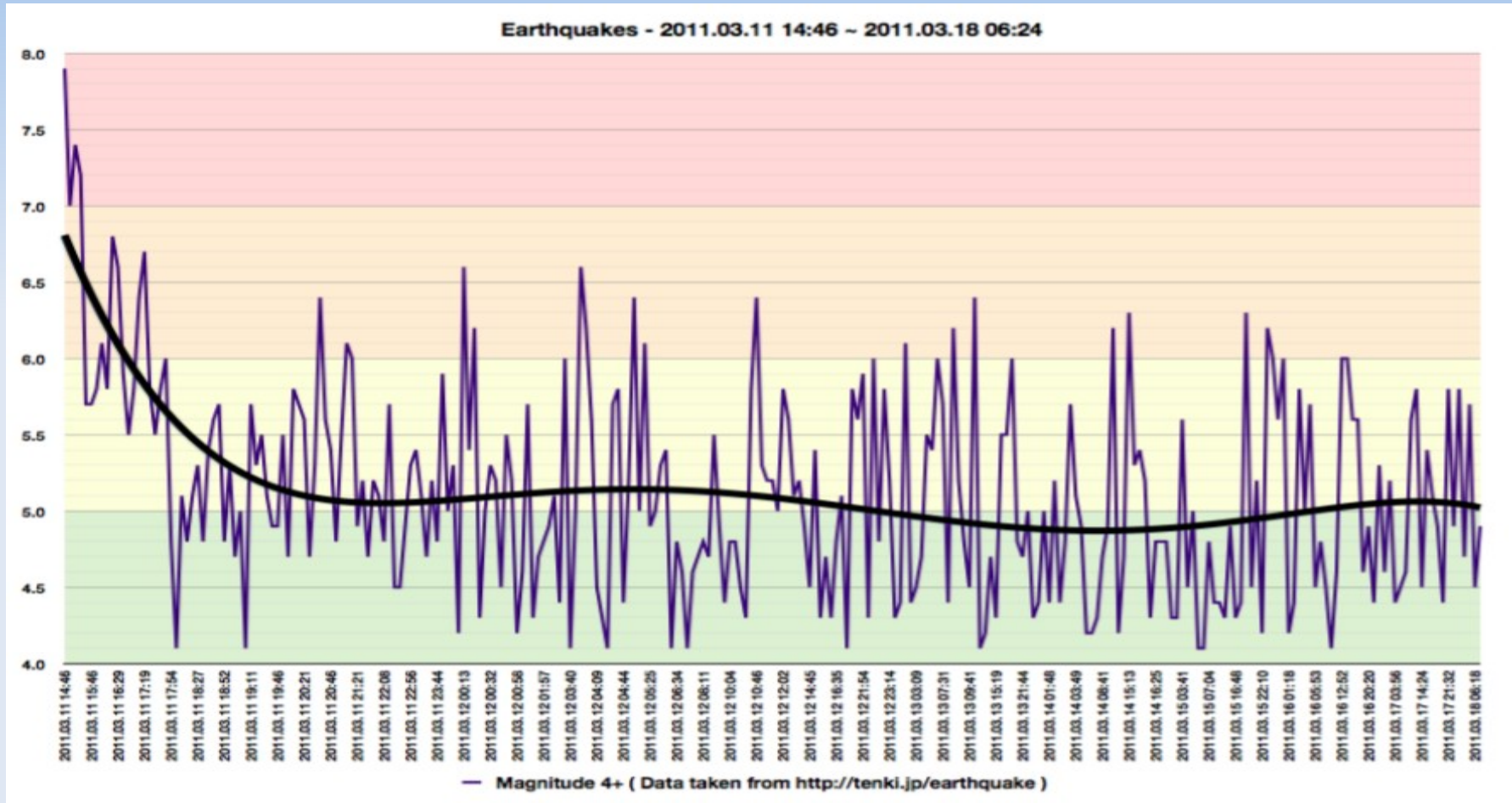


Chronologie eines Unfalls



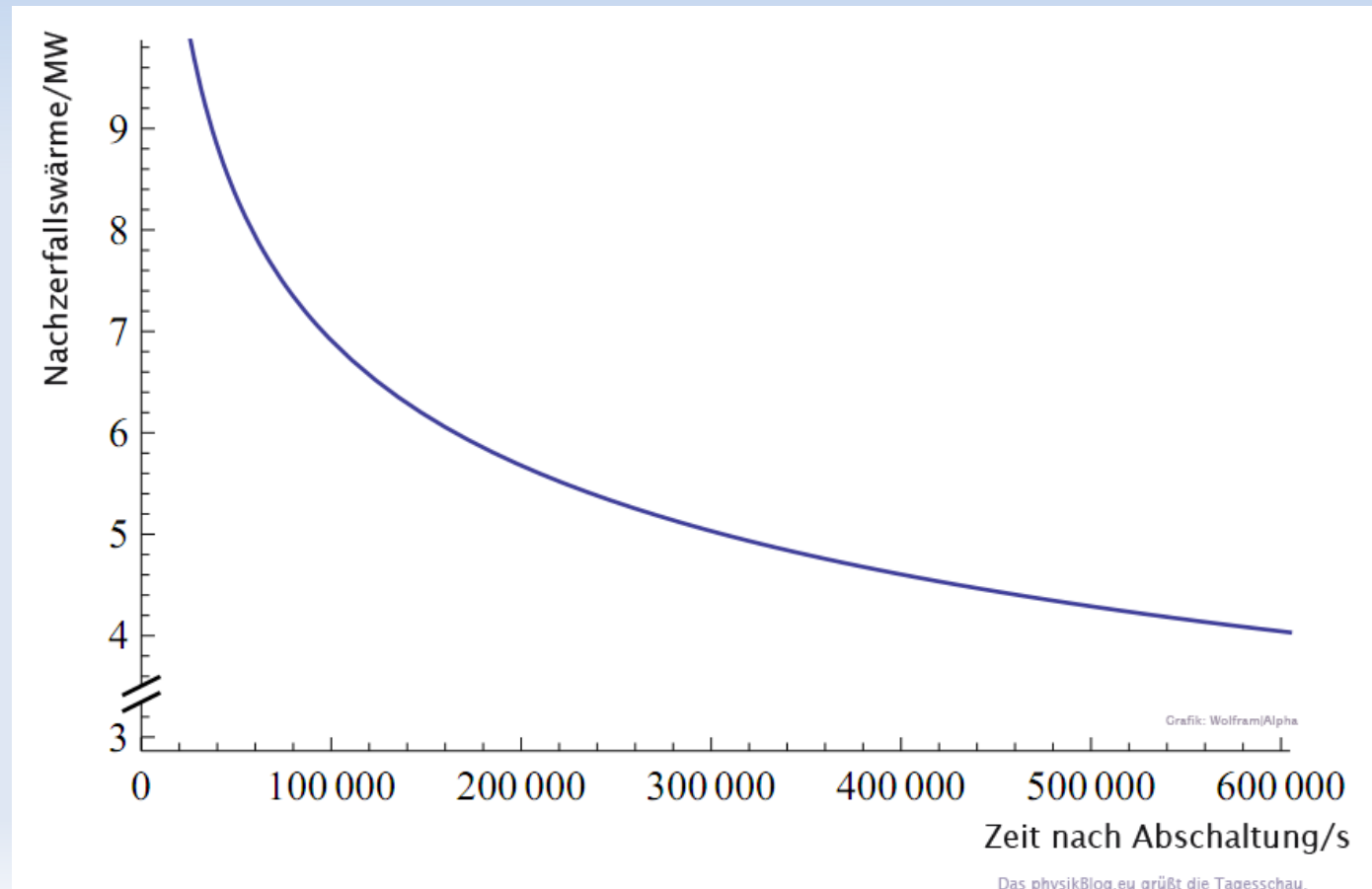
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Chronologie eines Unfalls



Chronologie eines Unfalls

$$\frac{P}{P_0} = 6,22 \cdot 10^{-2} \cdot \left[t^{-0,2} - (T_0 + t)^{-0,2} \right]$$



Chronologie eines Unfalls

Temp. [K]	Ereignis
3120	Schmelzpunkt von UO_2
2960	Schmelzpunkt von ZrO_2
2620	Schmelzpunkt von B_4C
2400 - 2600	Zerstörung der Brennstäbe
2100	Beg. Verflüssigung UO_2 - Zry
2030	Schmelzpunkt von Zry
1850	<i>Eskalation der Zry-Oxidation</i>
1700	Schmelzpunkt von Edelstahl
1450	Eutektika Zry - Ag, Zerstörung DWR-Steuerstäbe
1420	Eutektika Stahl - B_4C , Zerstörung SWR-Steuerstäbe
1270	<i>verstärkte Zry-Oxidation</i>
1210	Eutektika Stahl - Zr, relevant für DWR-Steuerstäbe
1170	Bersten von Brennstäben, Be- ginn Spaltproduktfreisetzung
1100	Schmelzpunkt von Ag-In-Cd

Chronologie eines Unfalls

Melting Temperatures

UO₂
2850 °C

ZrO₂
2690 °C

B₄C
2450 °C

Zircaloy 4
1760 °C

Stainless Steel
1450 °C

Liquefaction Regimes

Melting of the ceramic materials UO₂ and ZrO₂ as well as formation of ceramic (U, Zr, O) melts

Melting of metallic Zircaloy and α-Zry(O) results in fast dissolution of UO₂

Start of rapid oxidation of Zircaloy by steam and macroscopic liquefaction by eutectic interaction of B₄C with stainless steel or stainless steel with Zircaloy

Ballooning and bursting of fuel rod claddings, release of volatile fission products

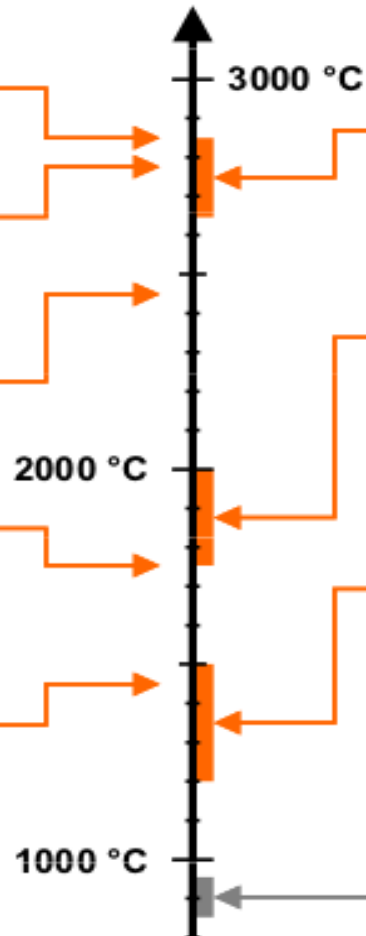
Core Damage

► Complete

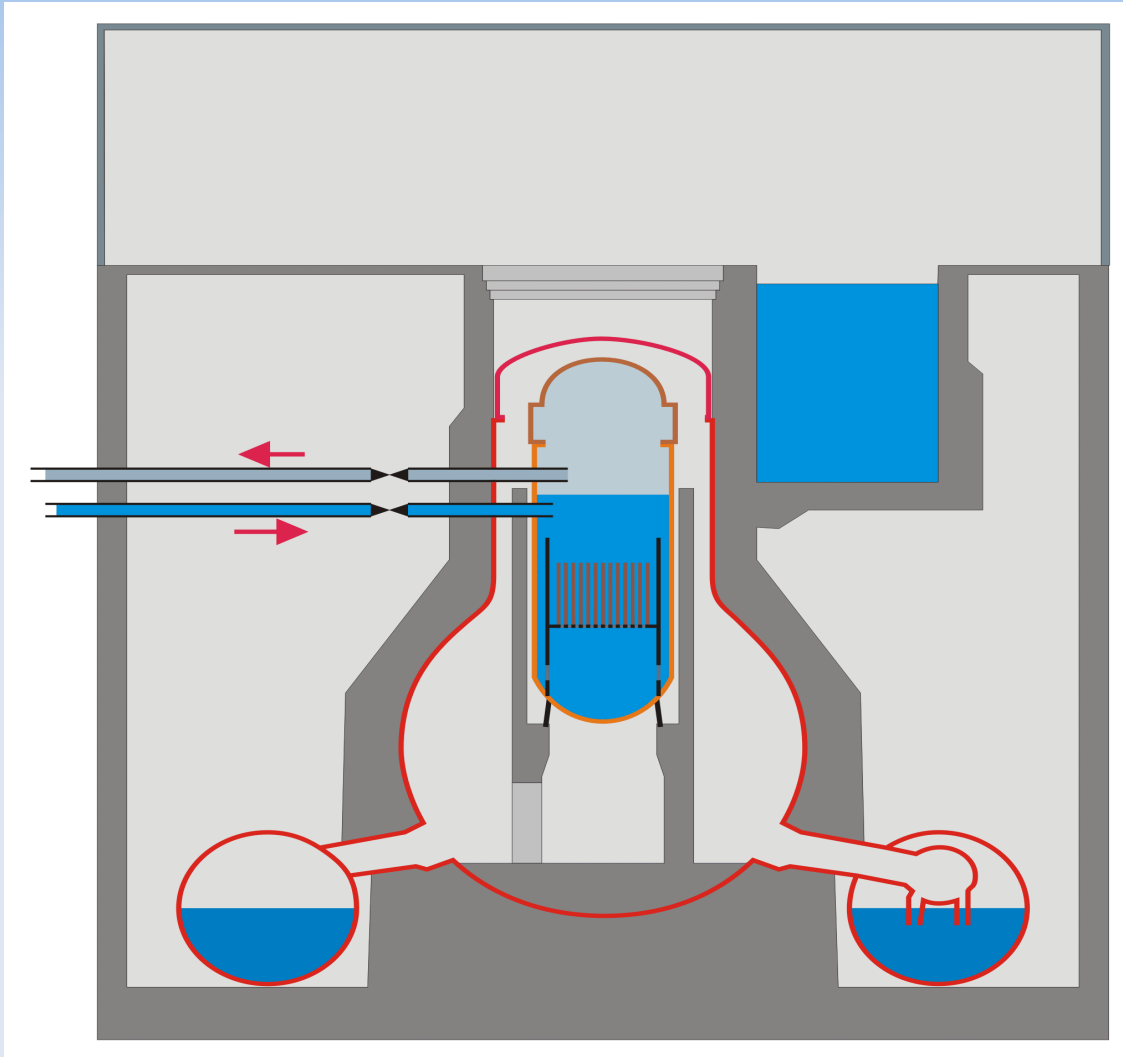
► Extended

► Localized

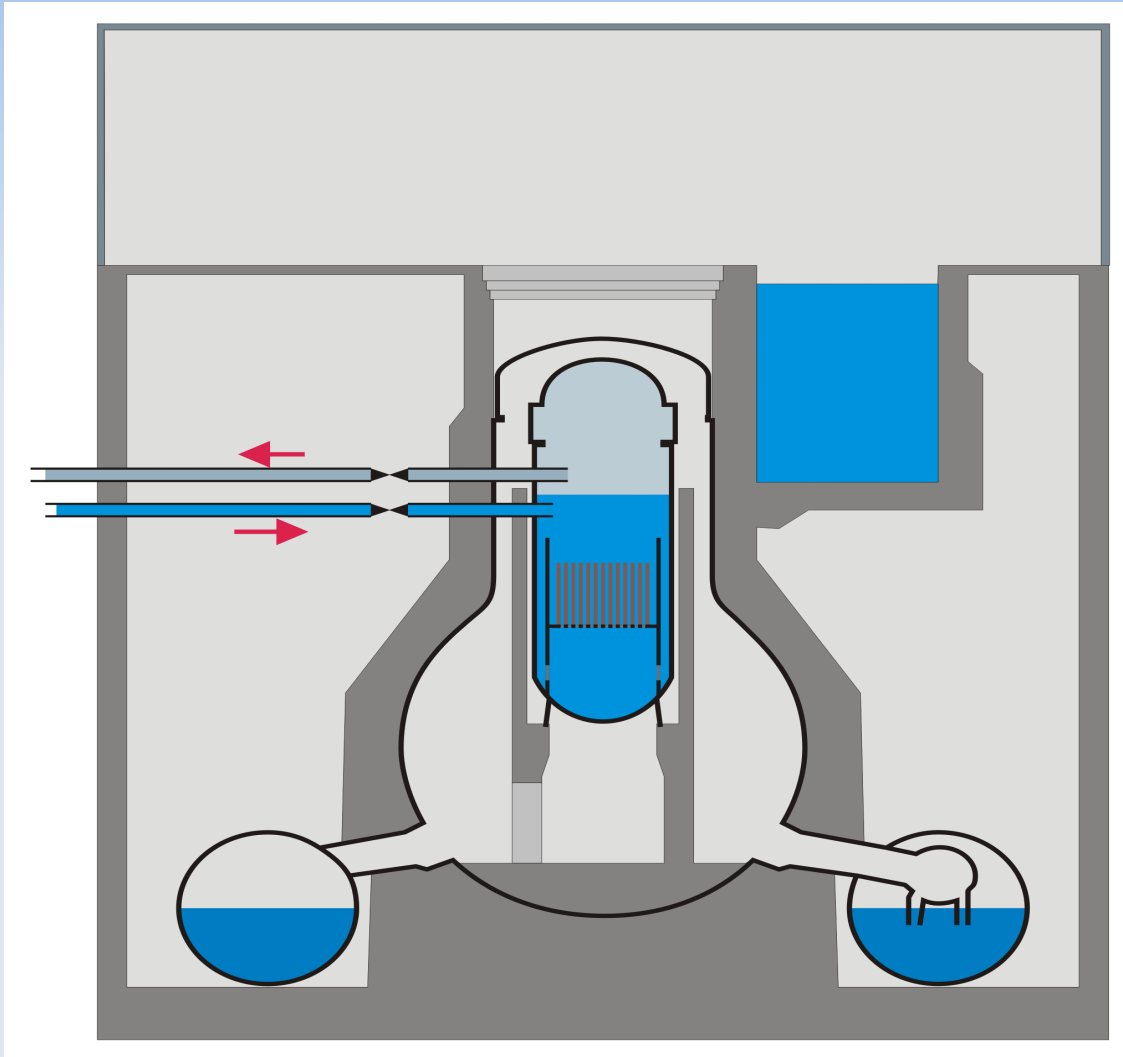
► Initiation



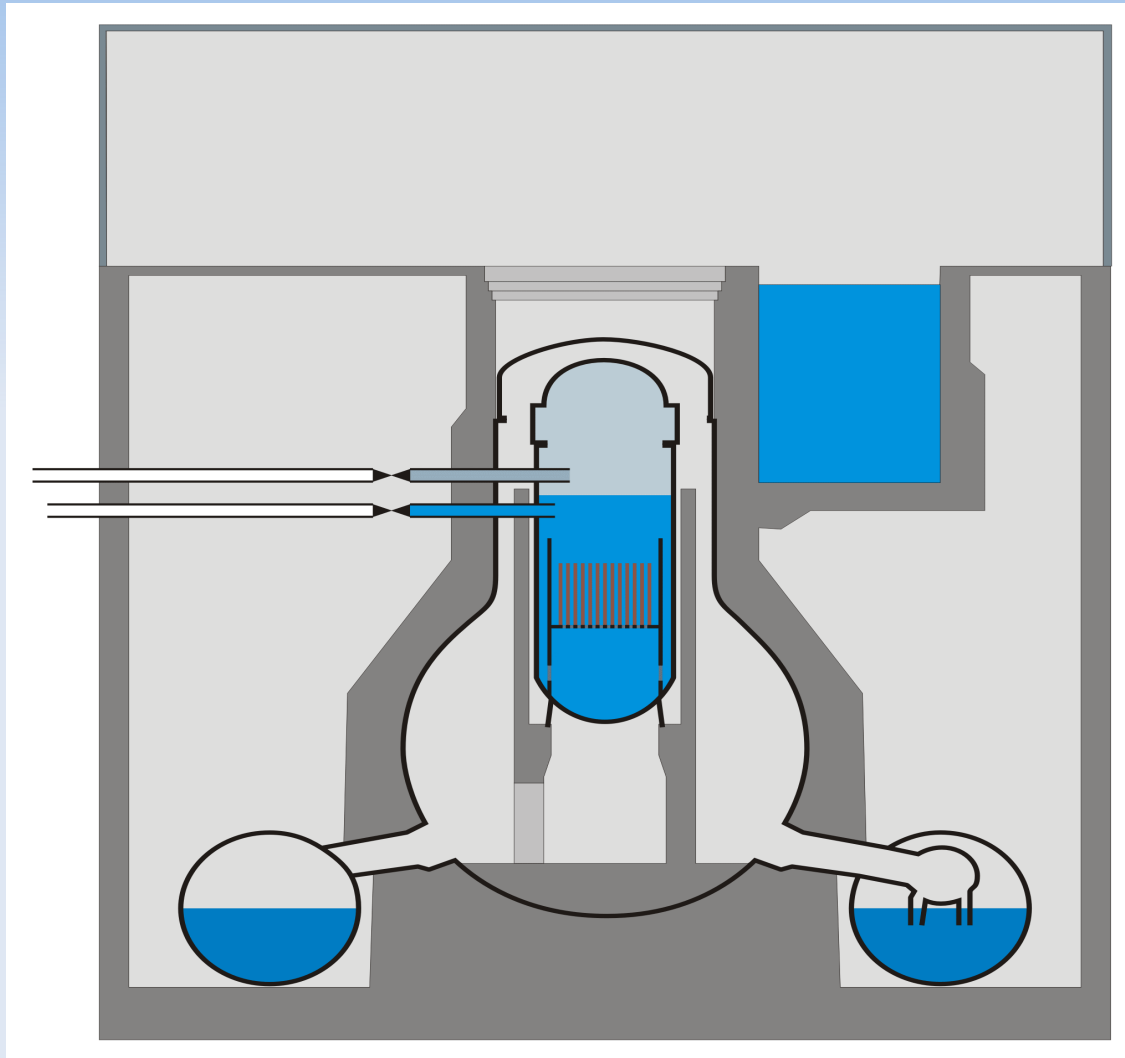
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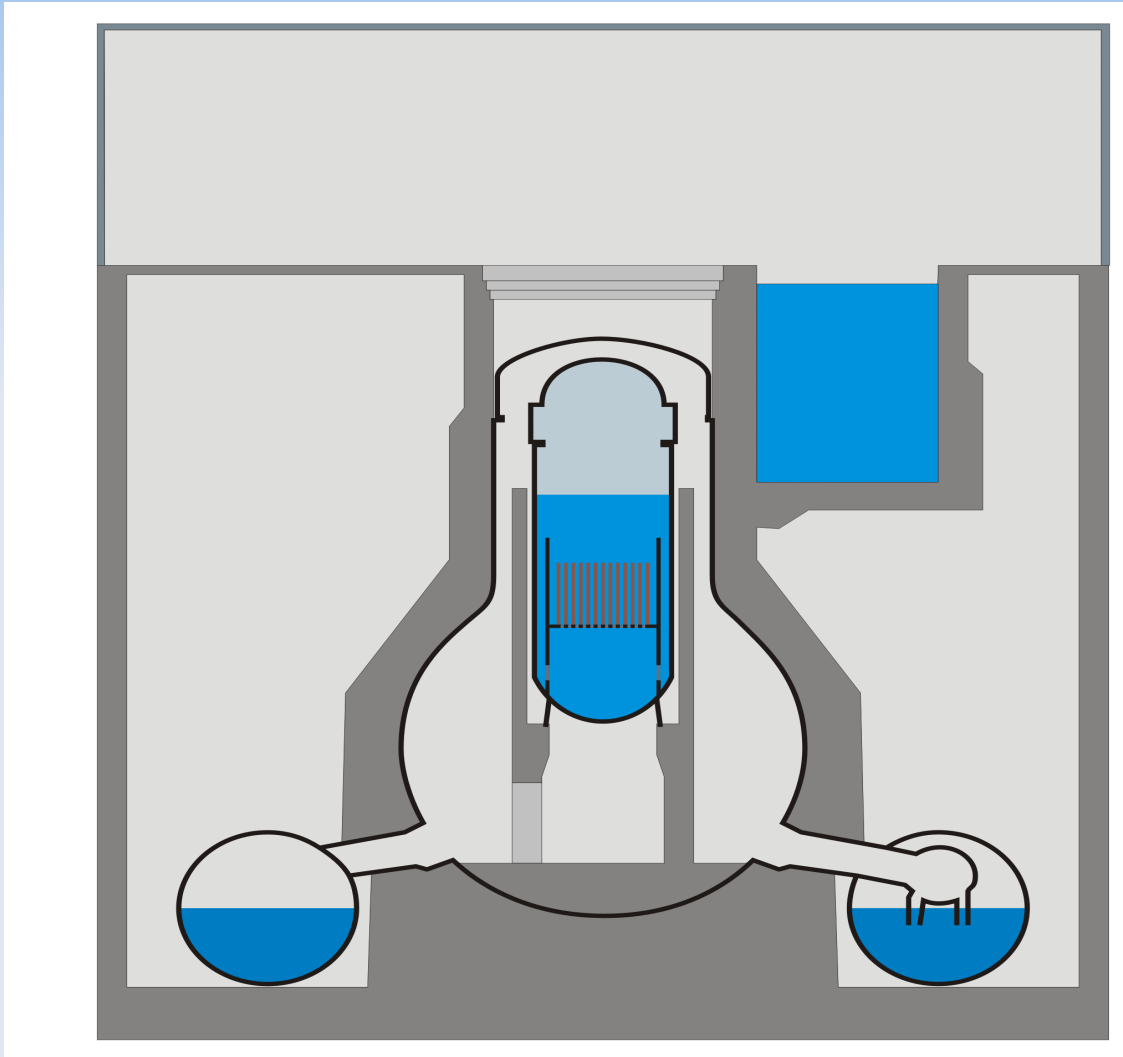
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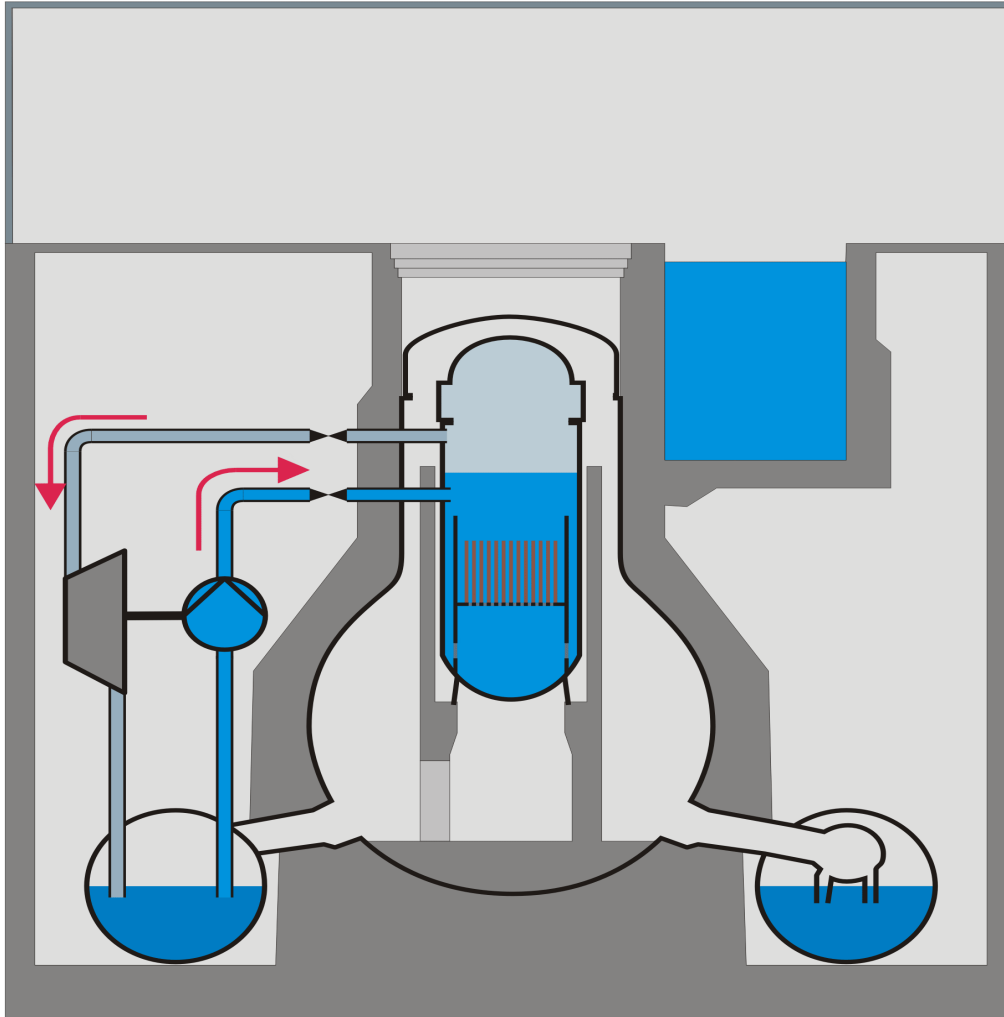
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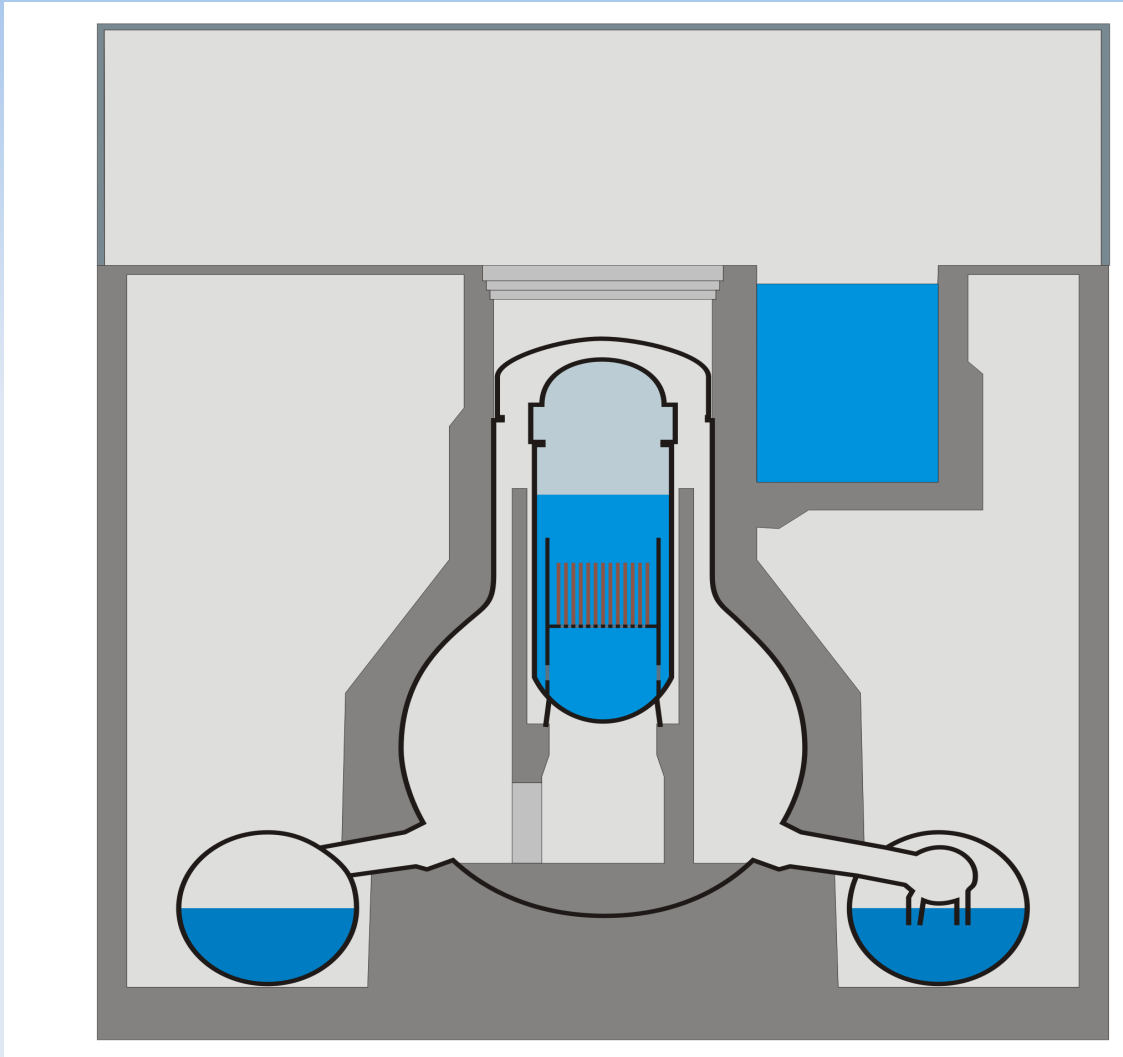
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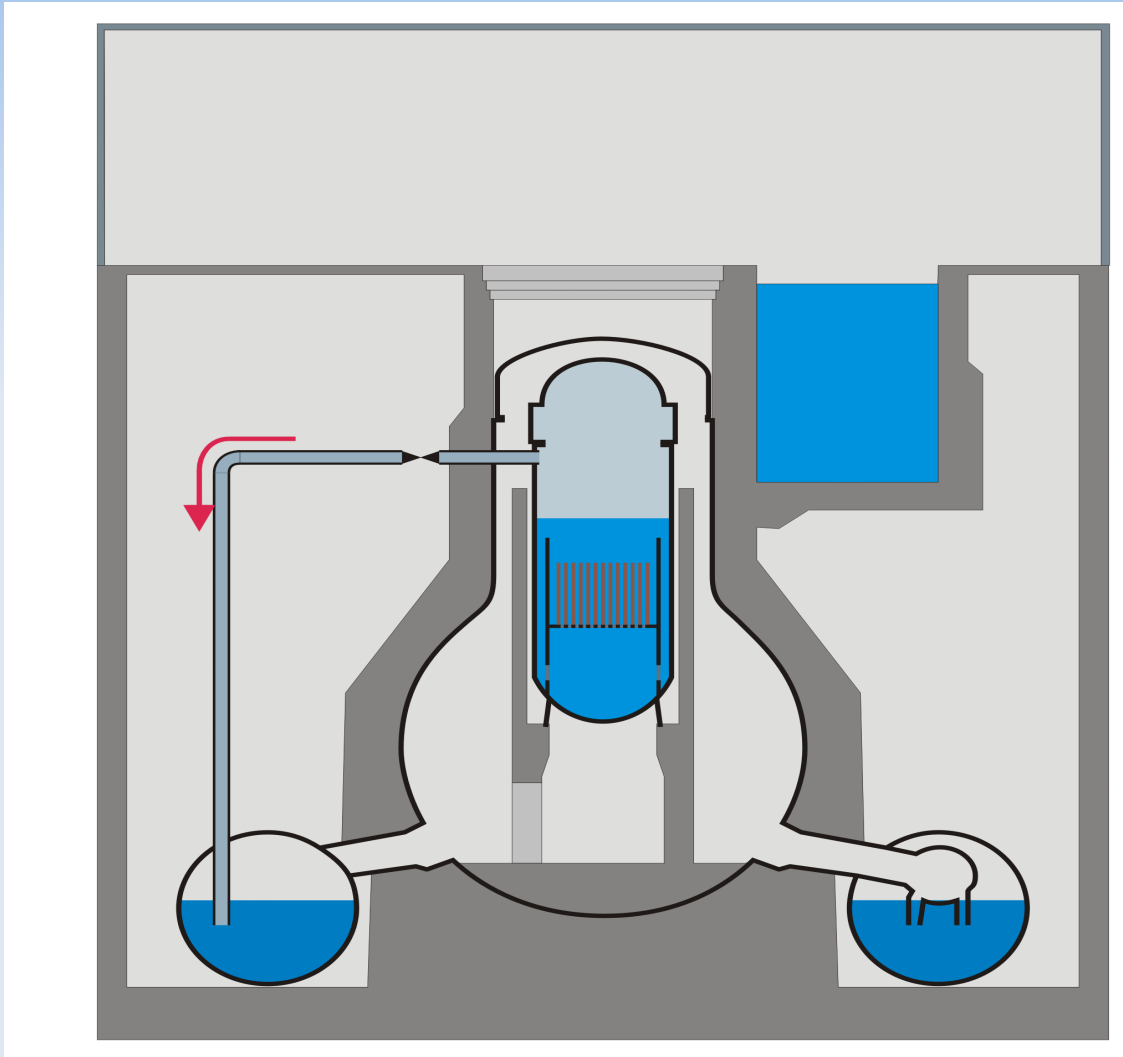
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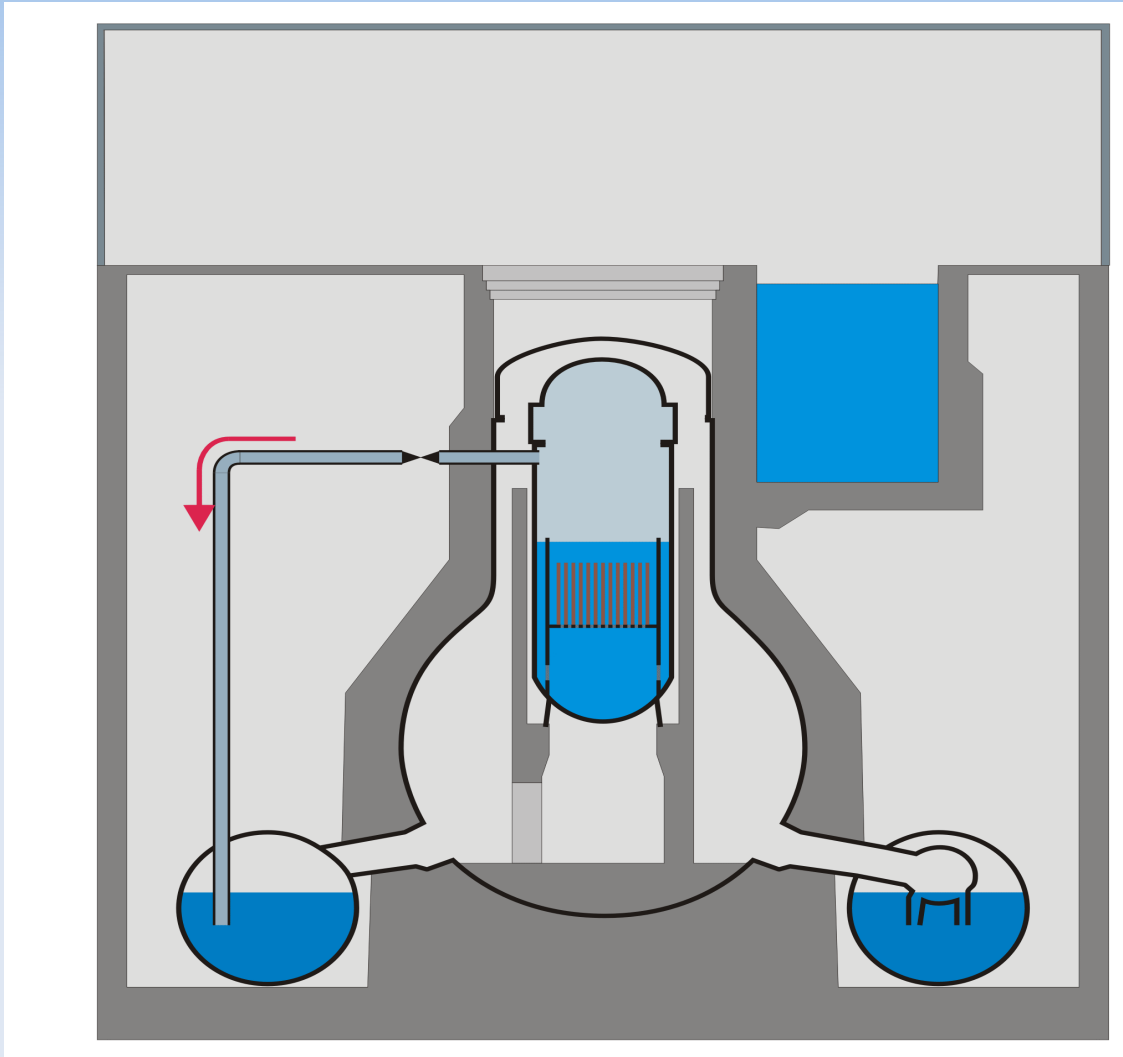
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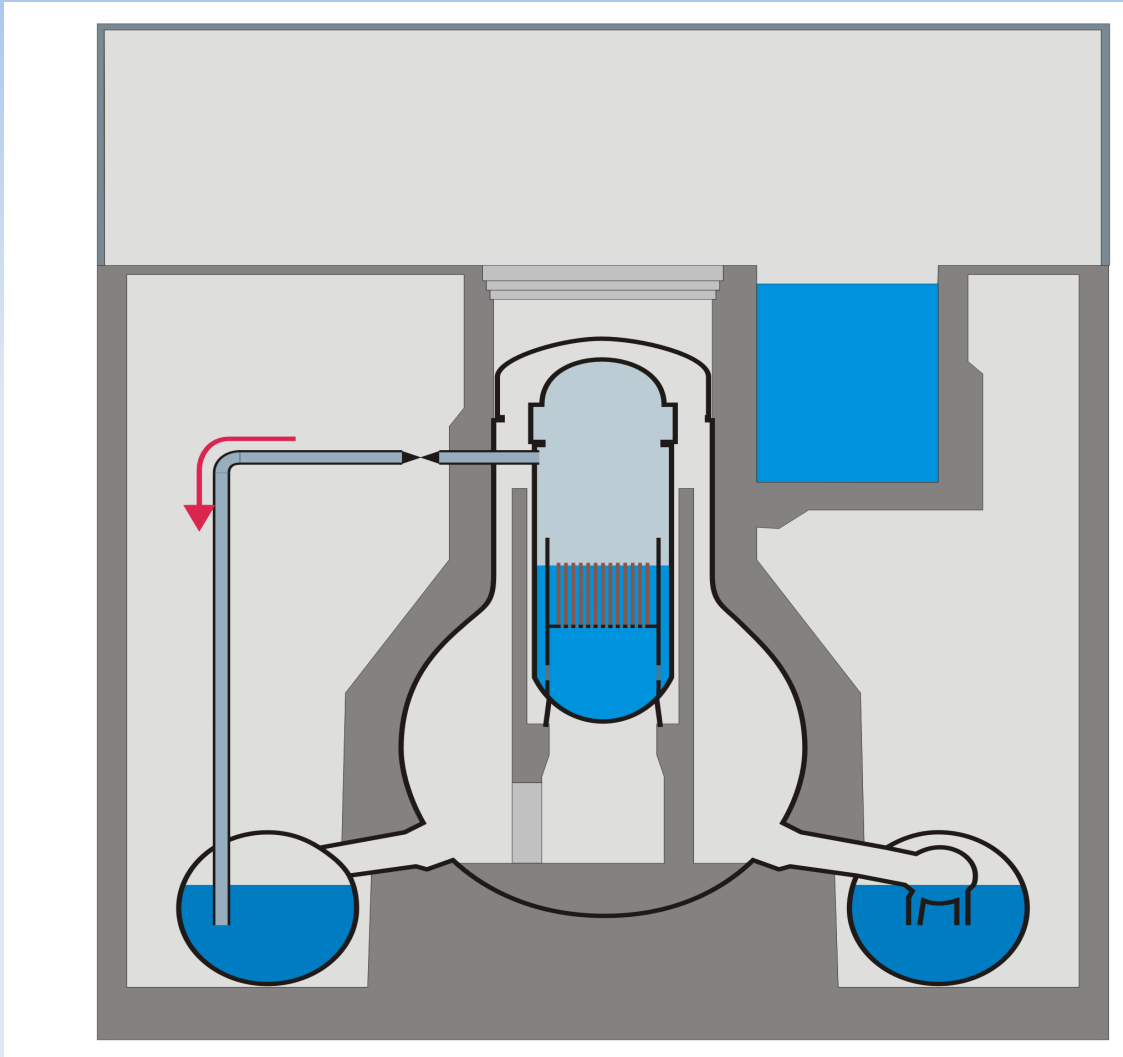
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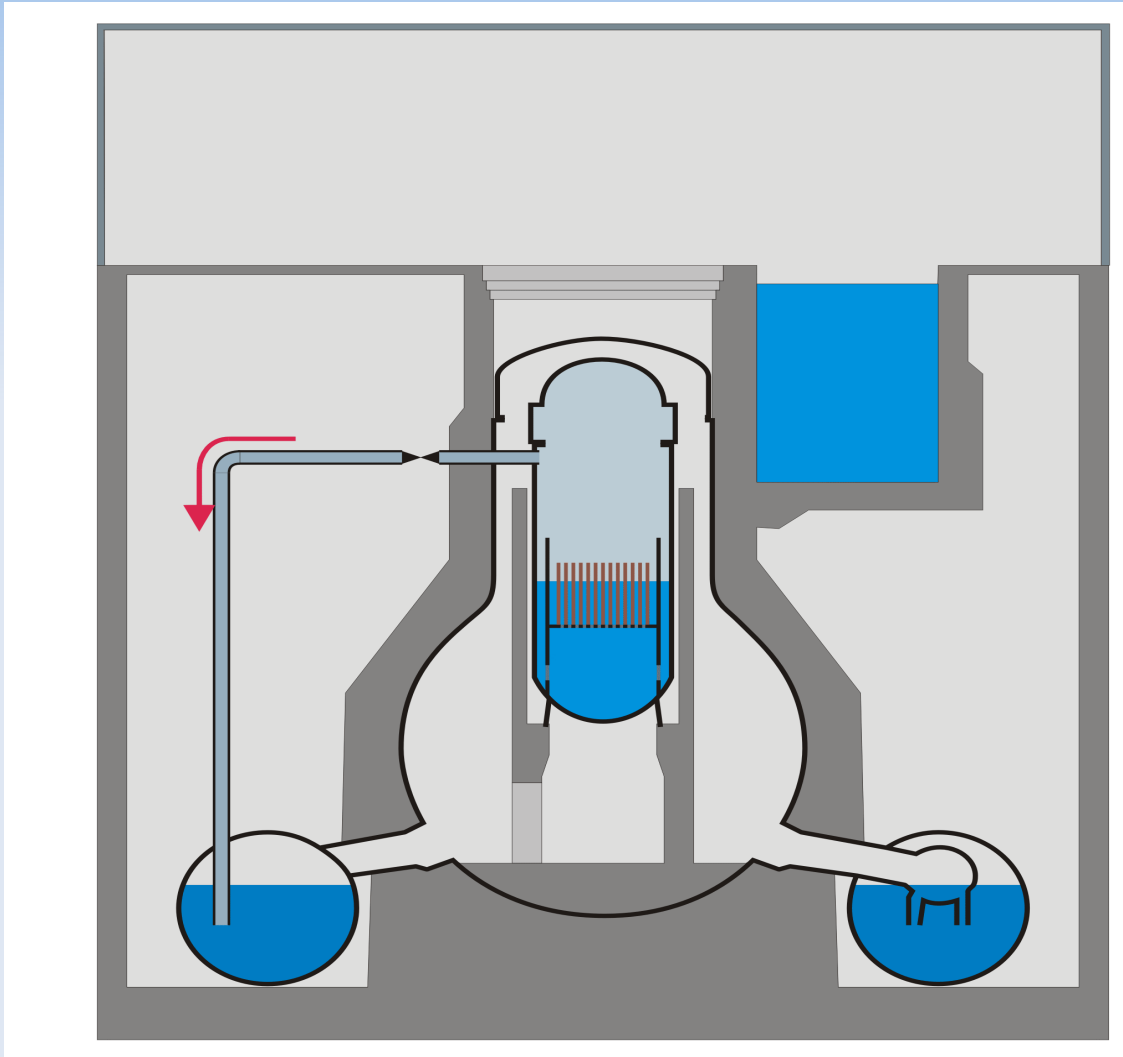
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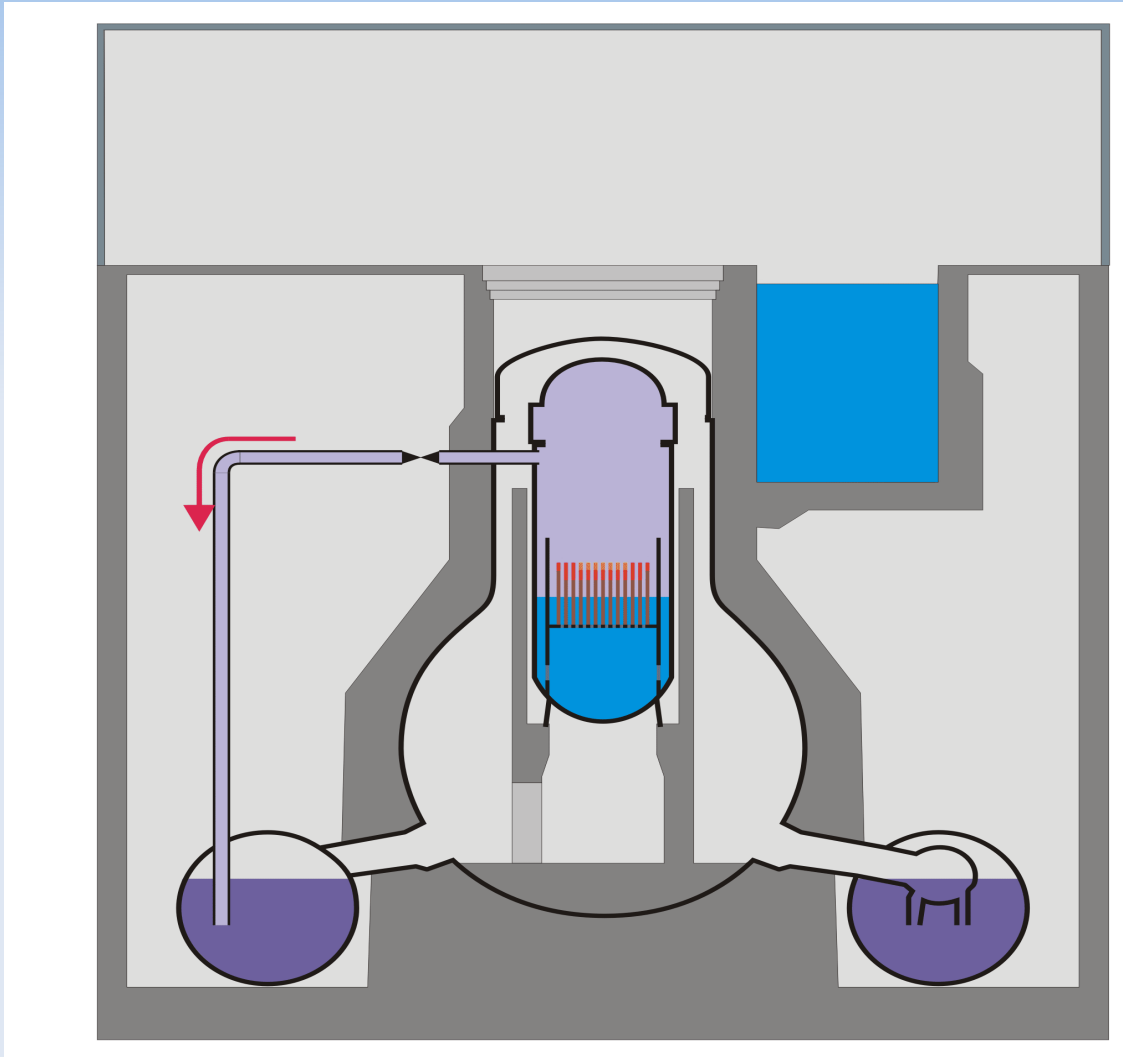
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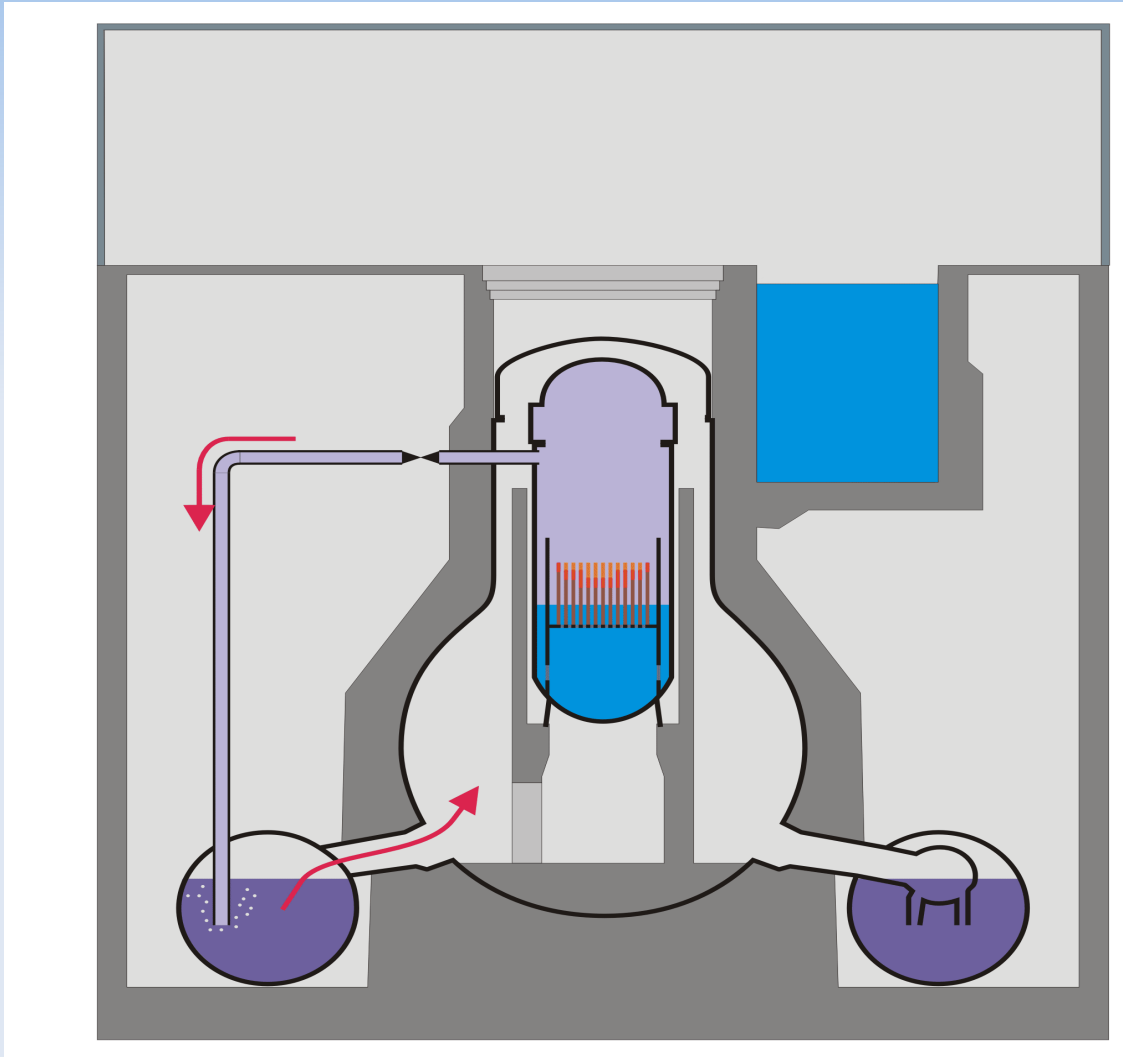
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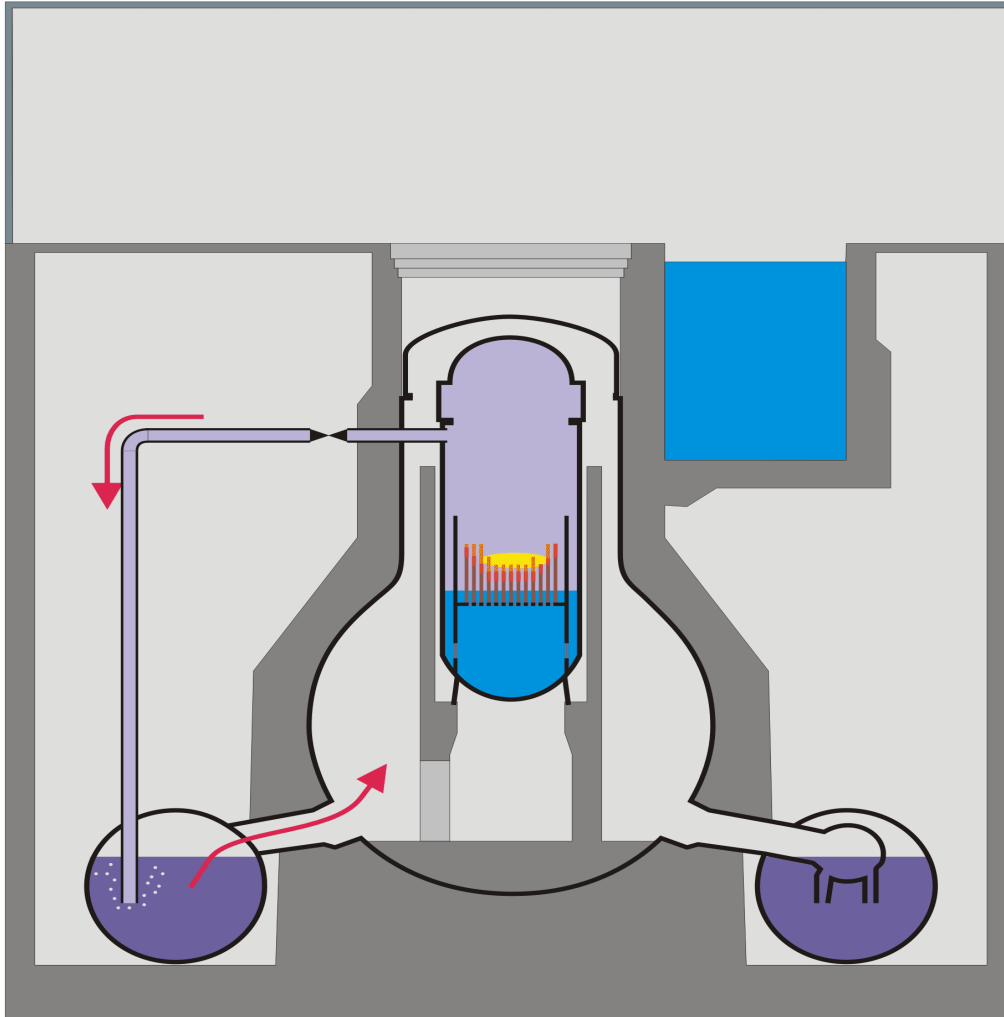
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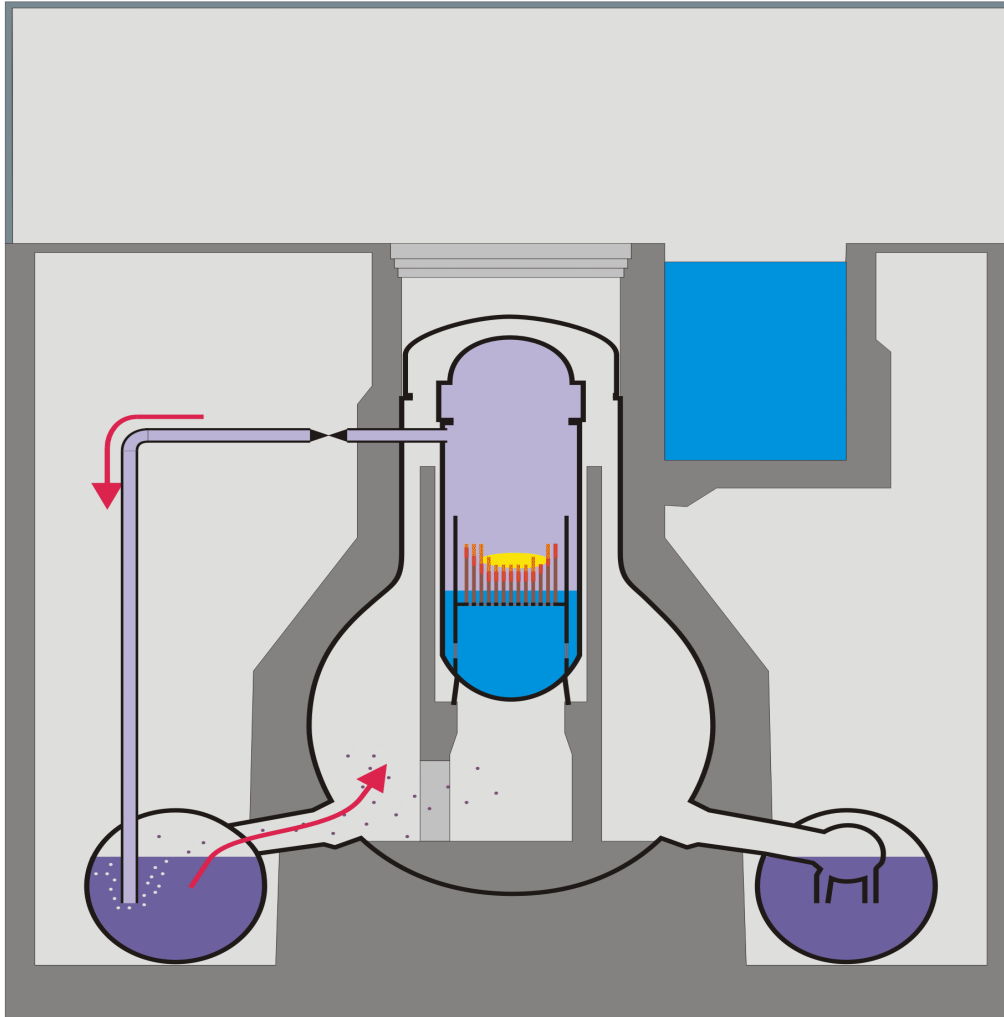
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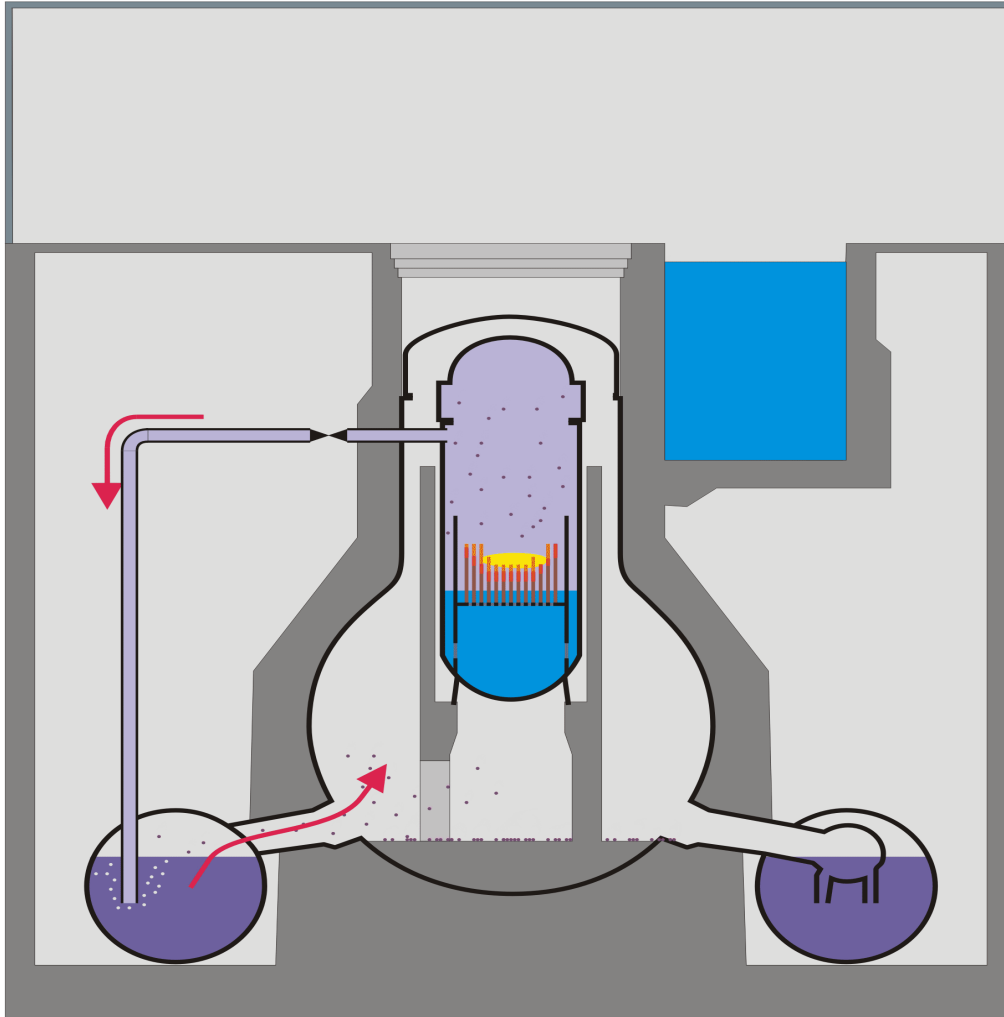
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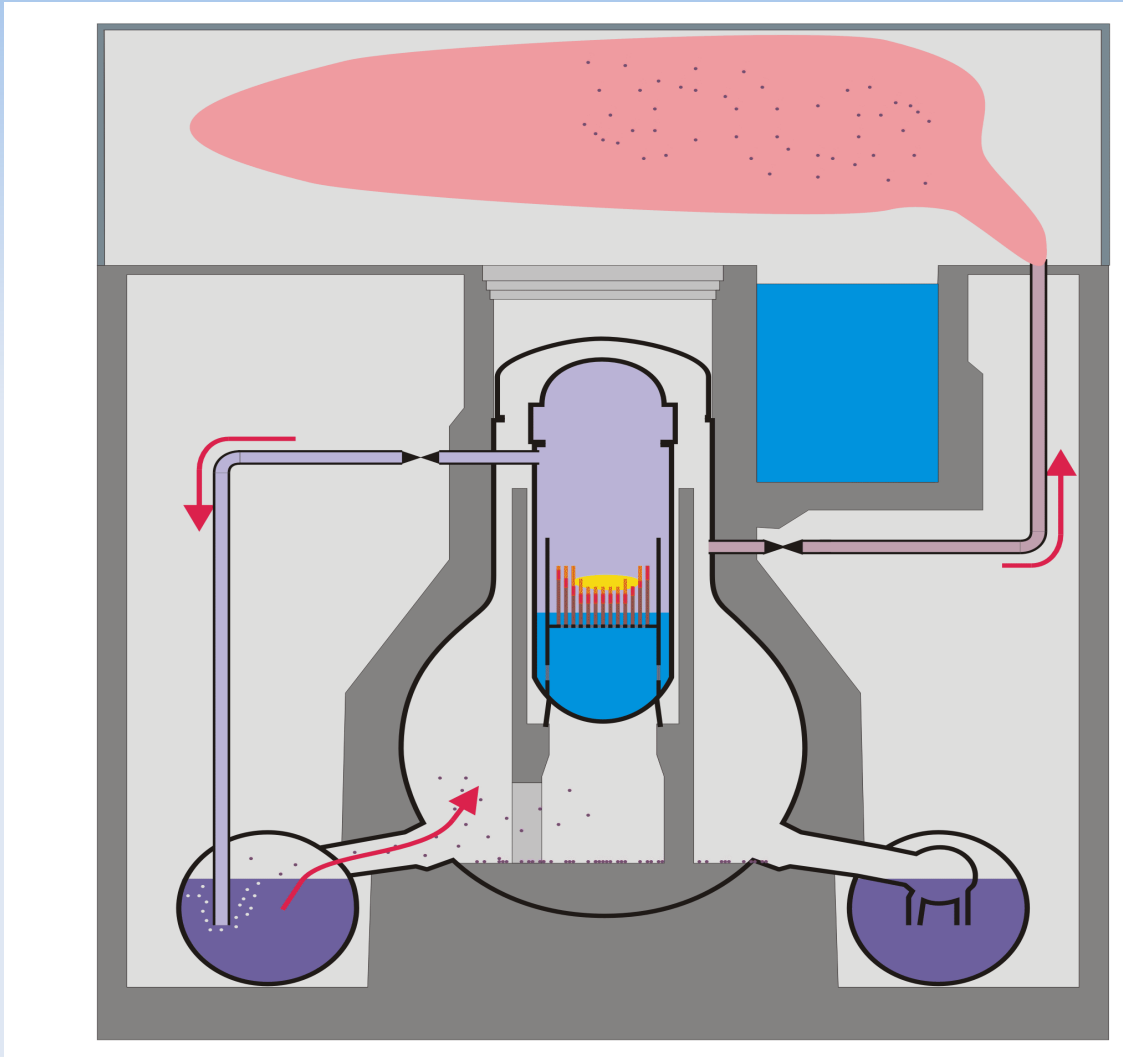
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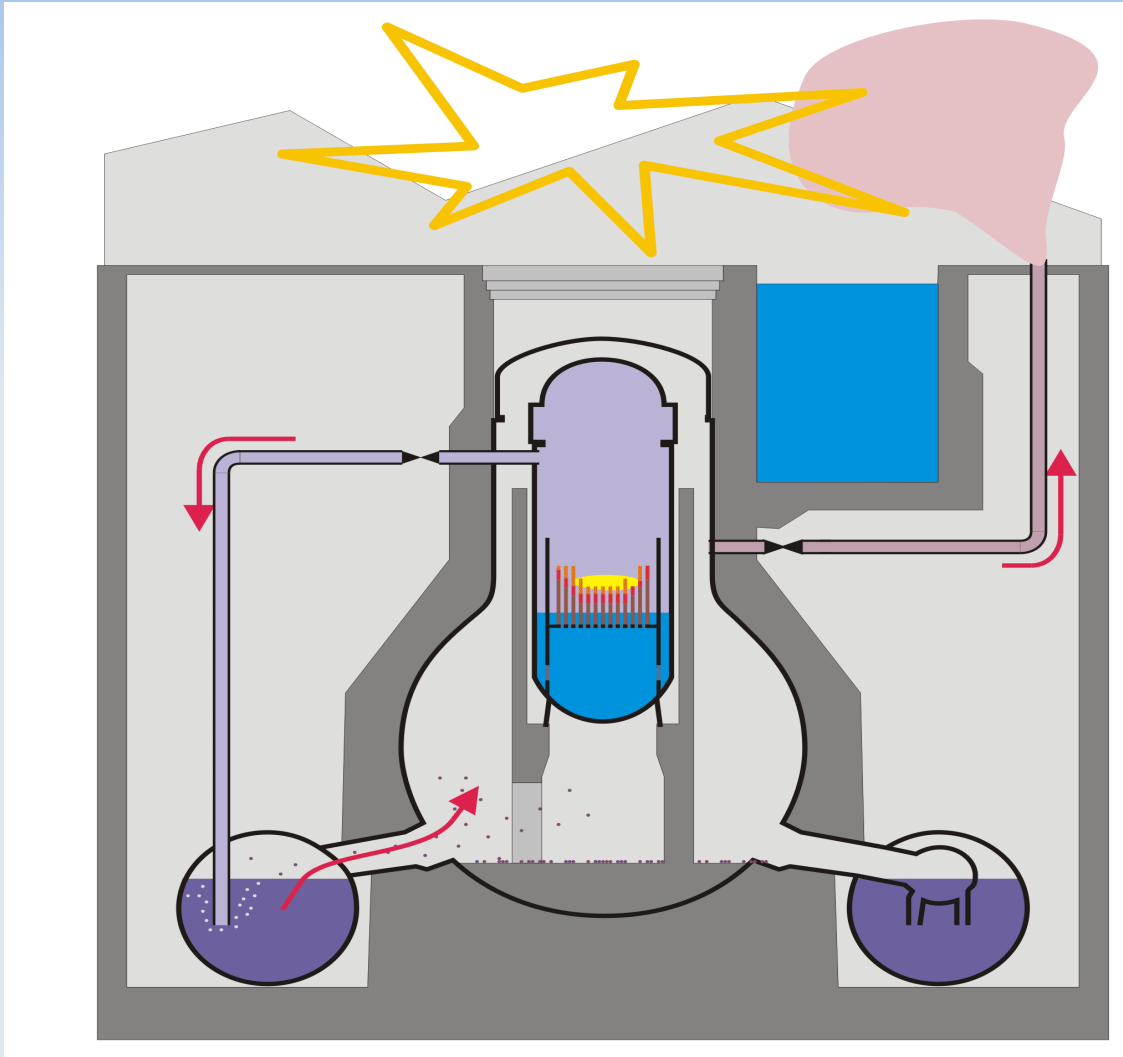
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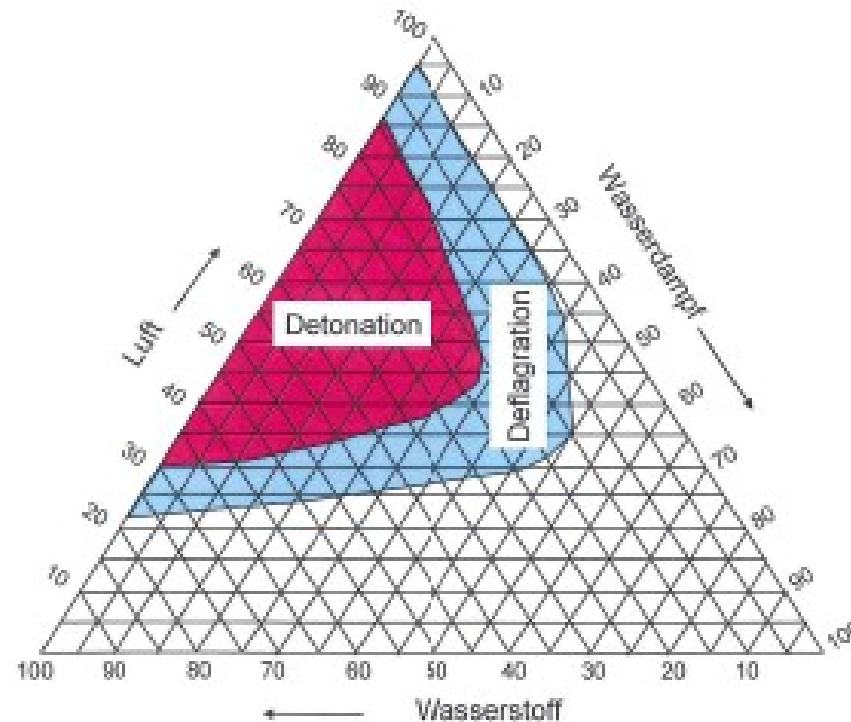
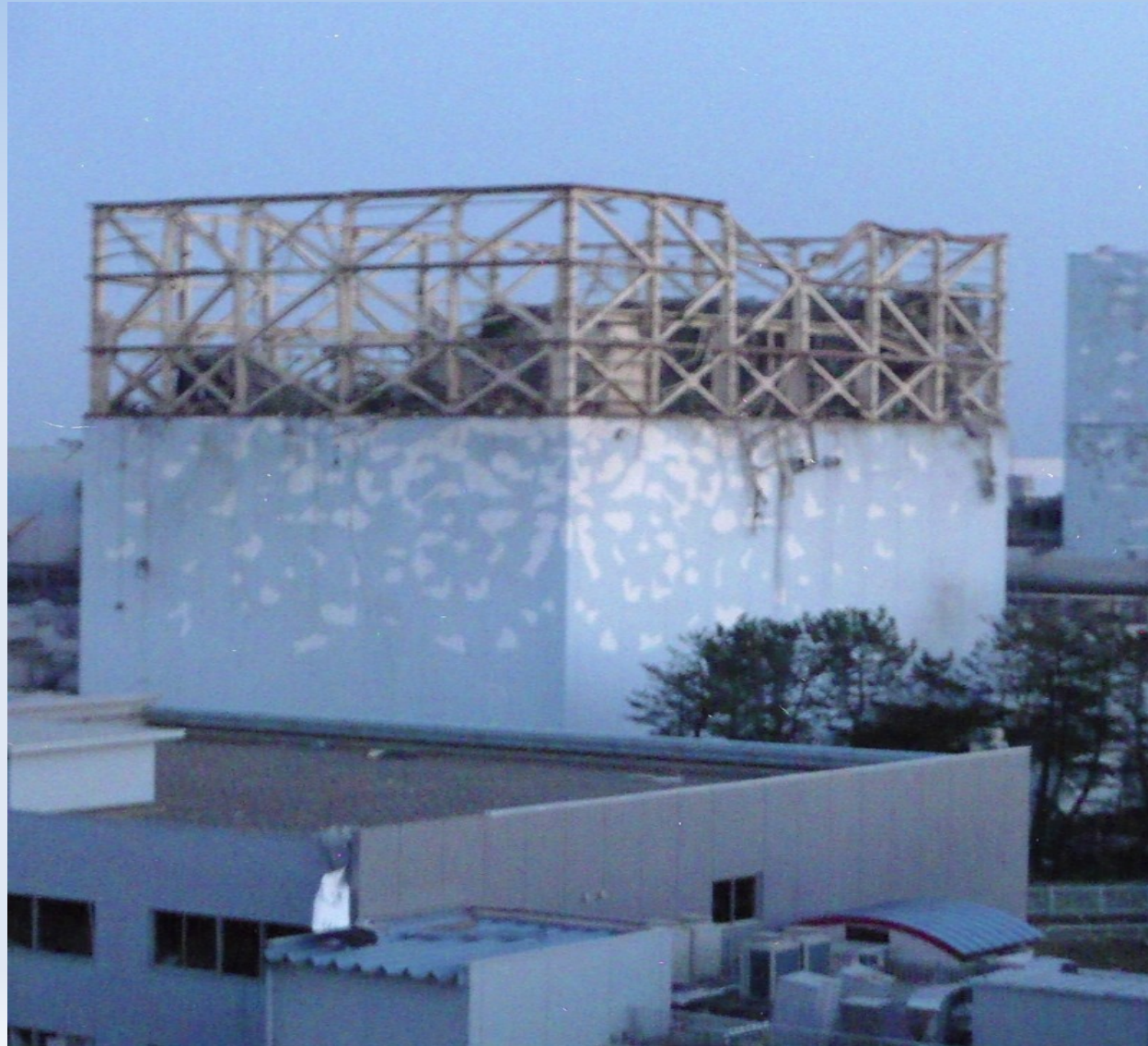
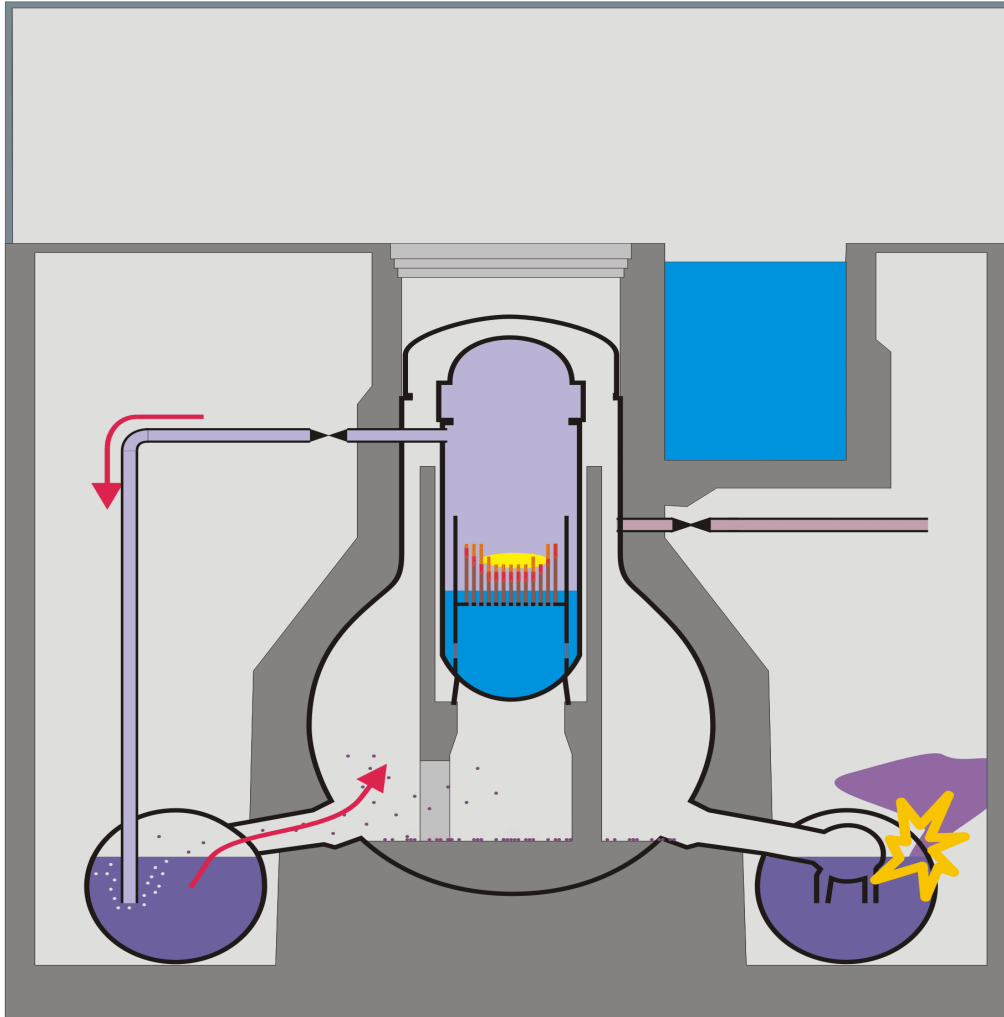


Abb. 23: Deflagrations- bzw. Detonationsbereich eines Wasserstoff-Luft-Wasserdampfgemischs

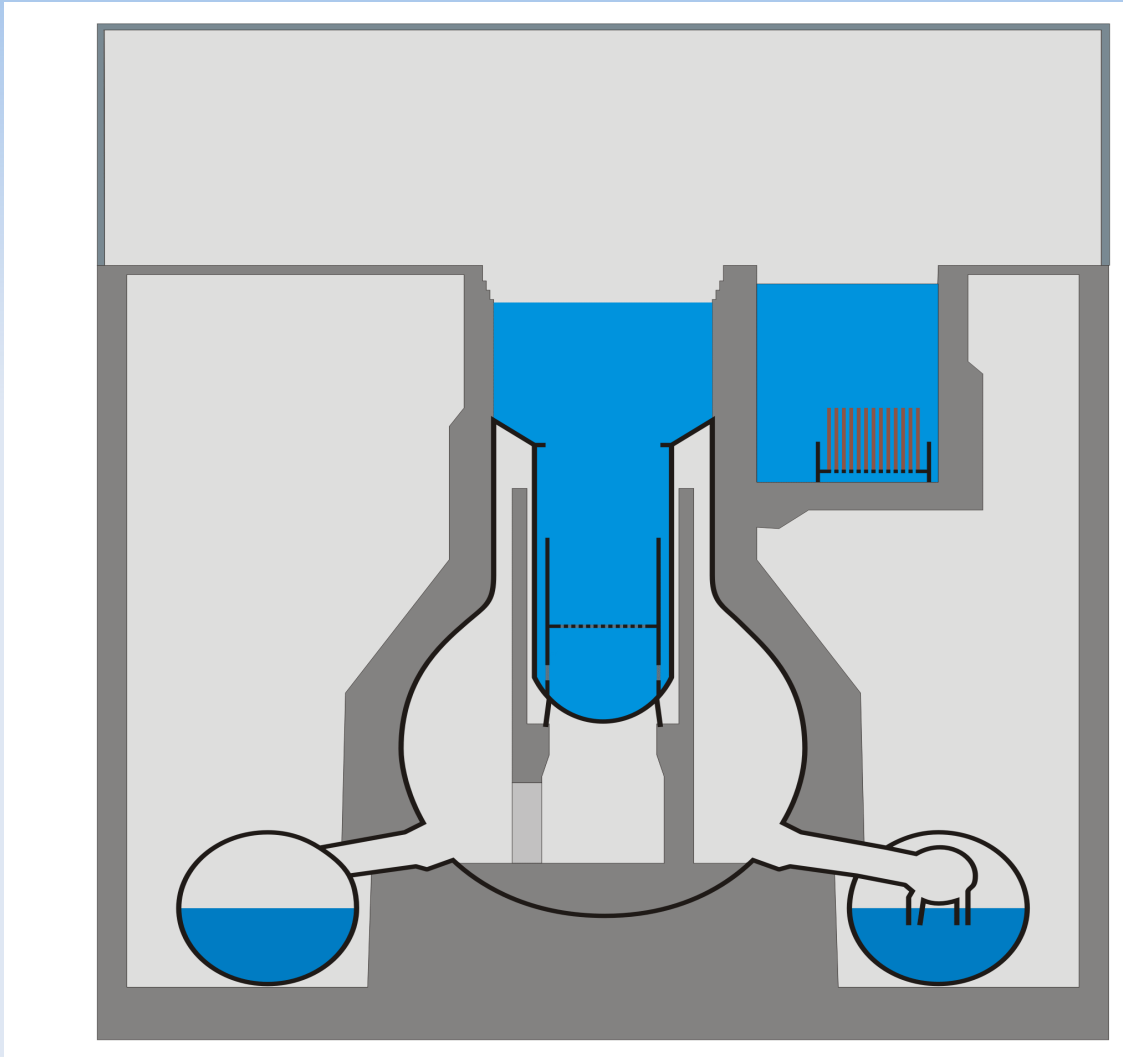
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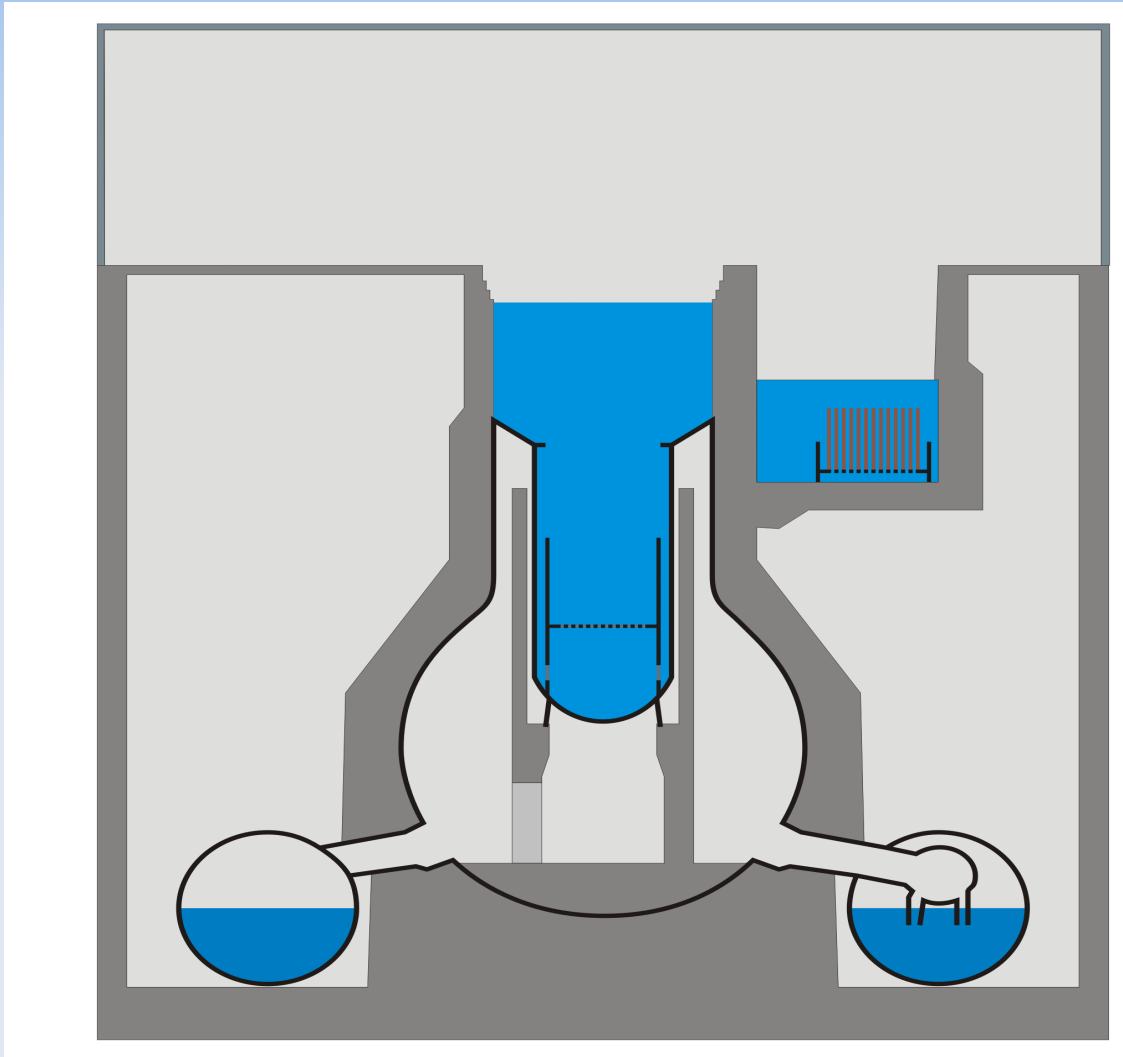
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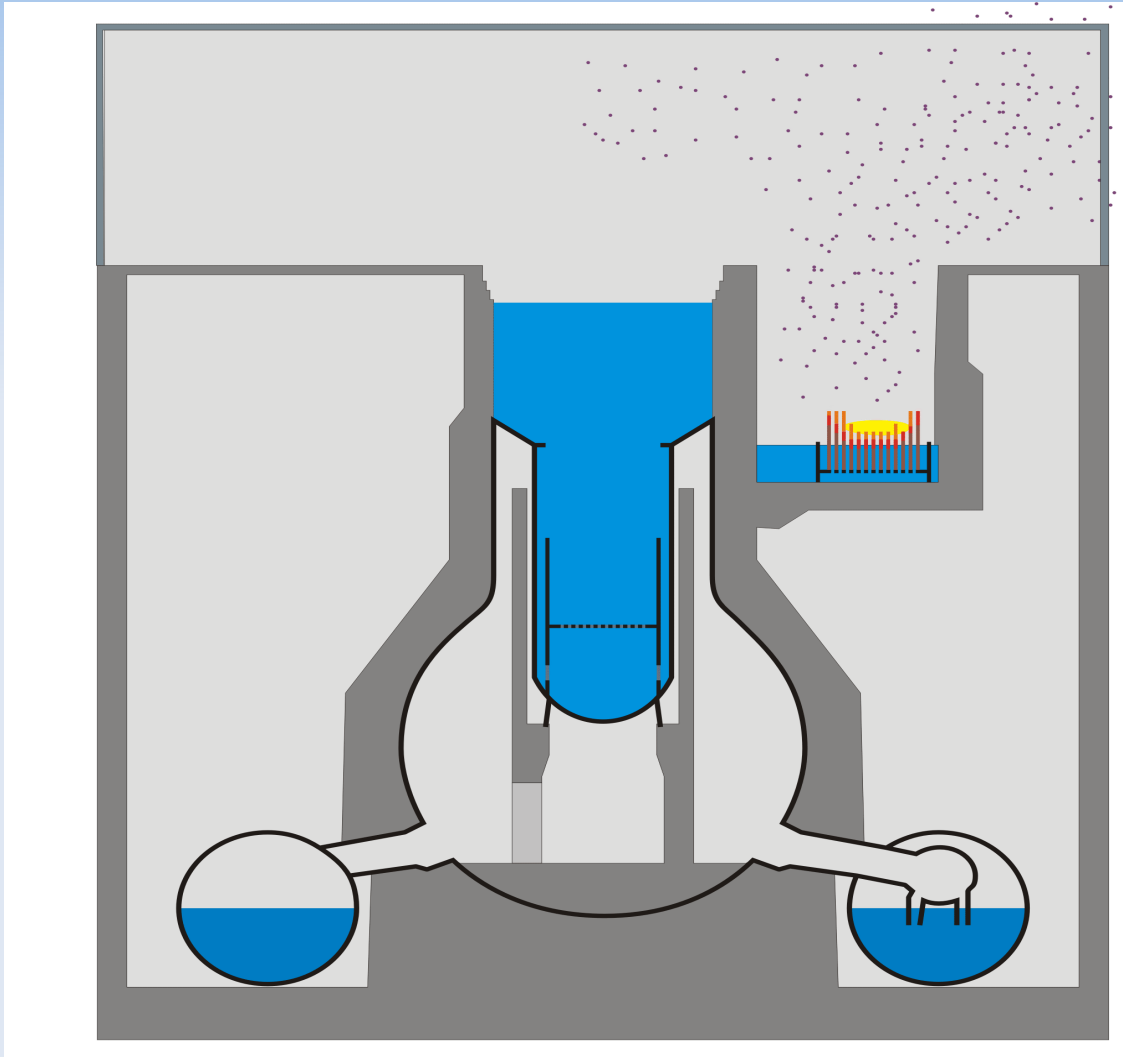
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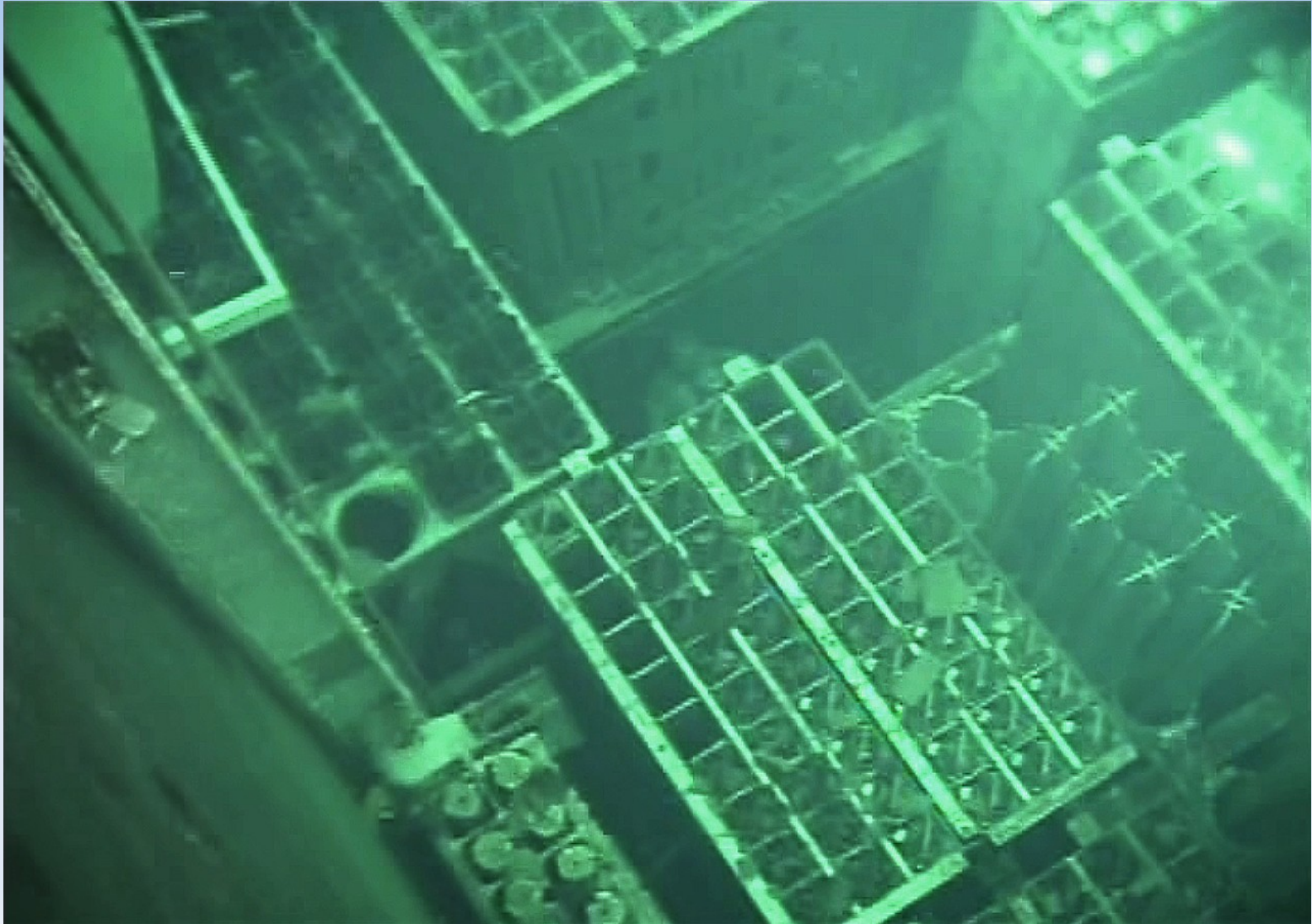
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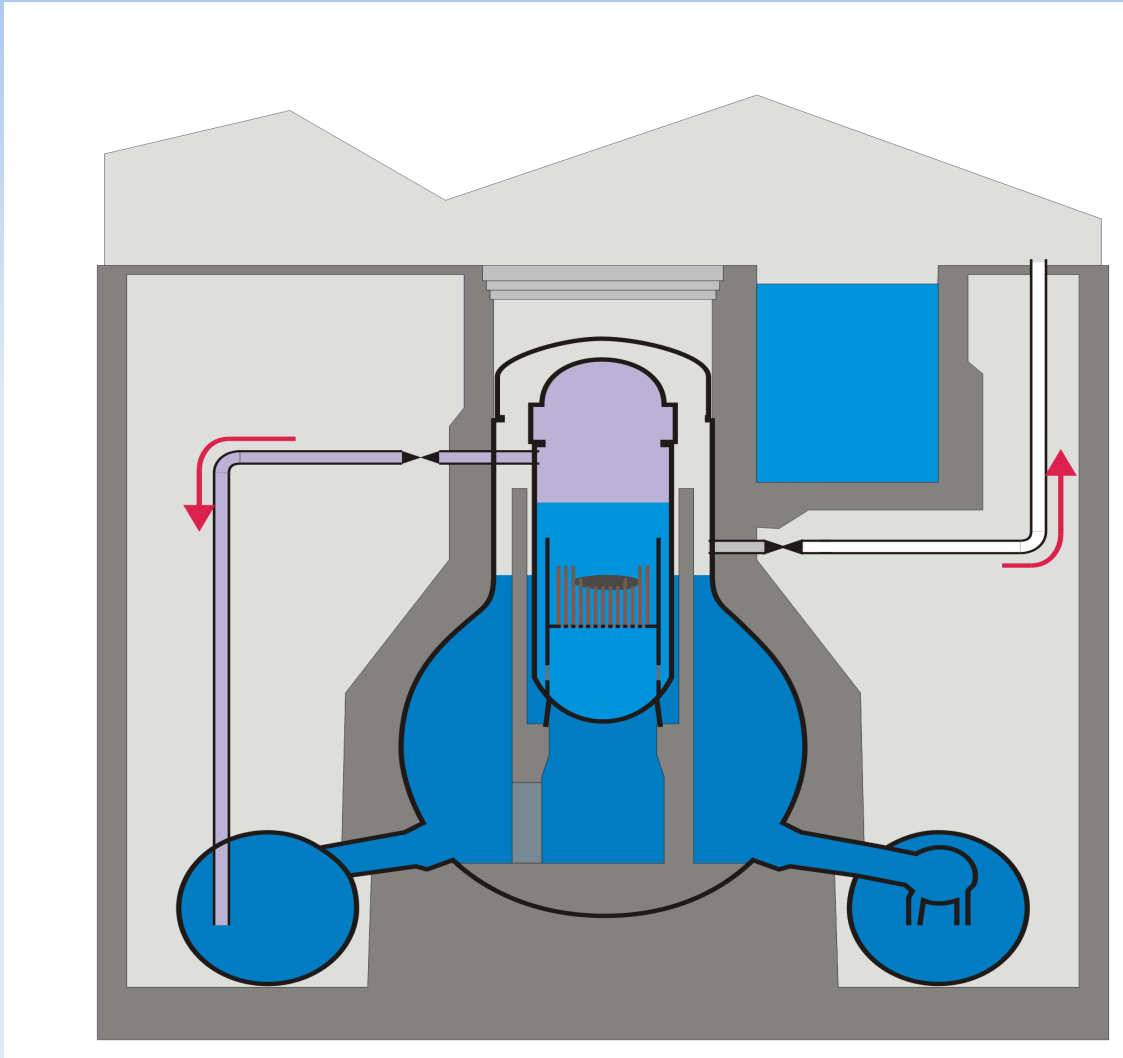
Chronologie eines Unfalls



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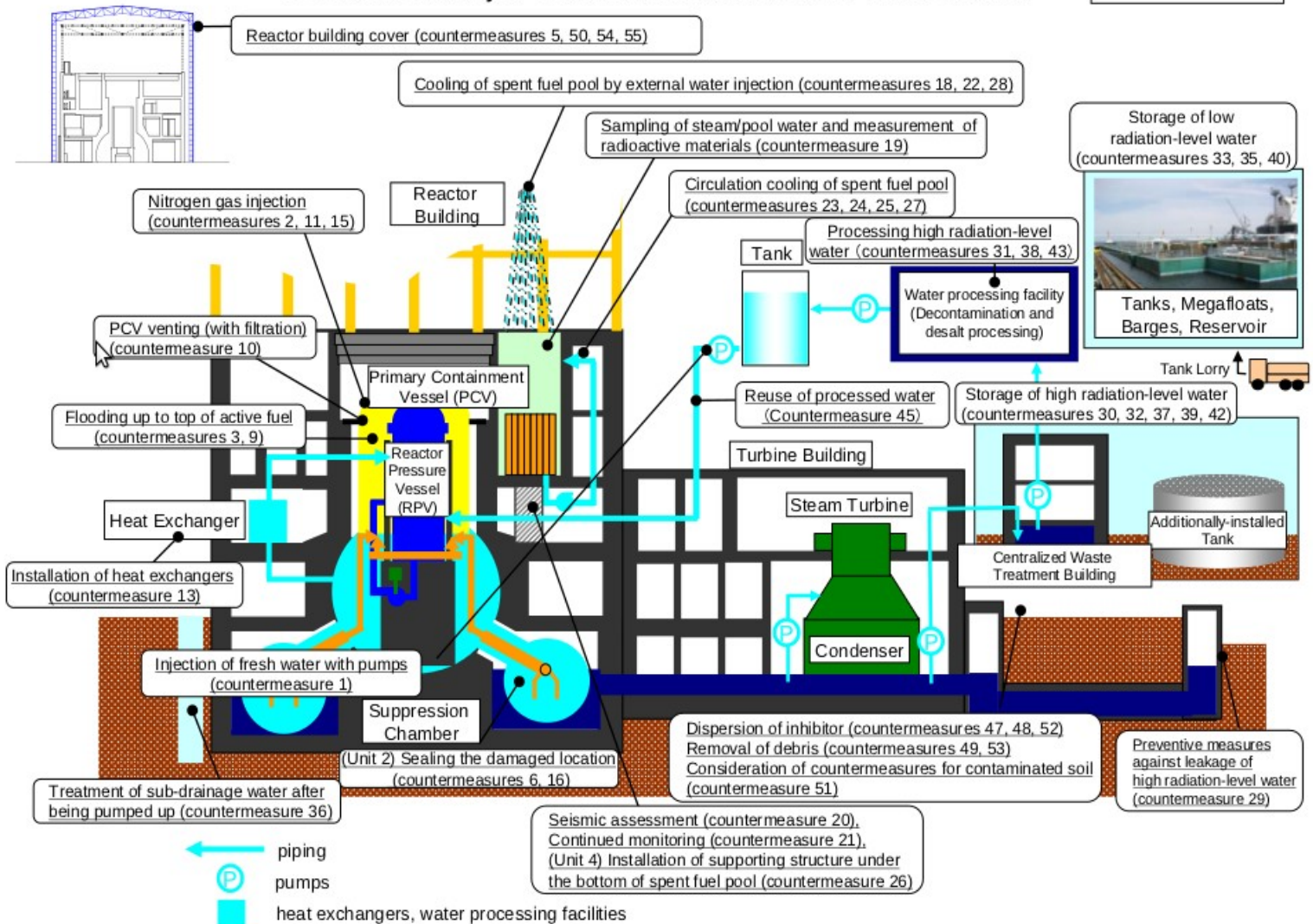
Chronologie eines Unfalls



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Overview of Major Countermeasures in the Power Station

Reference 2

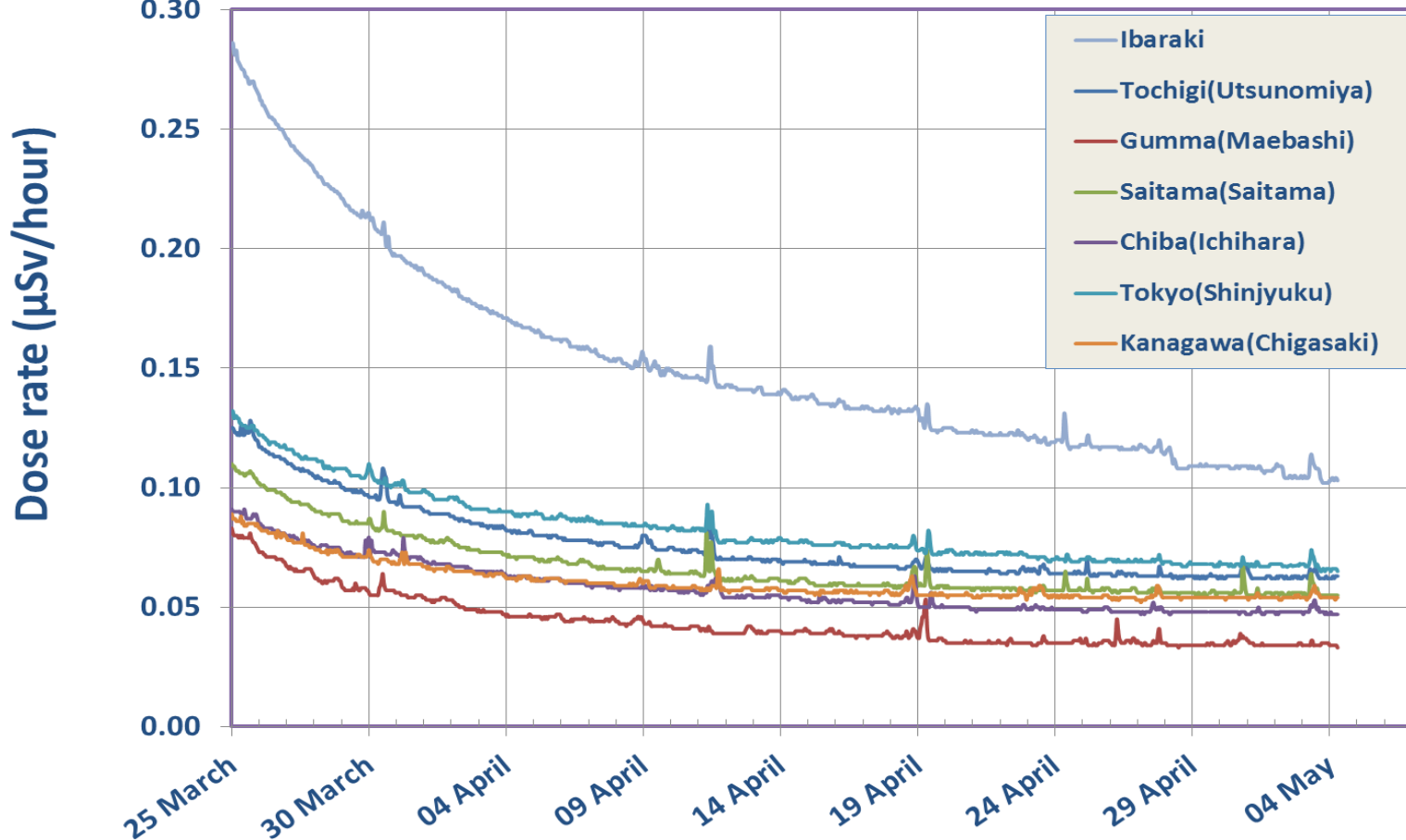


Maßnahmen



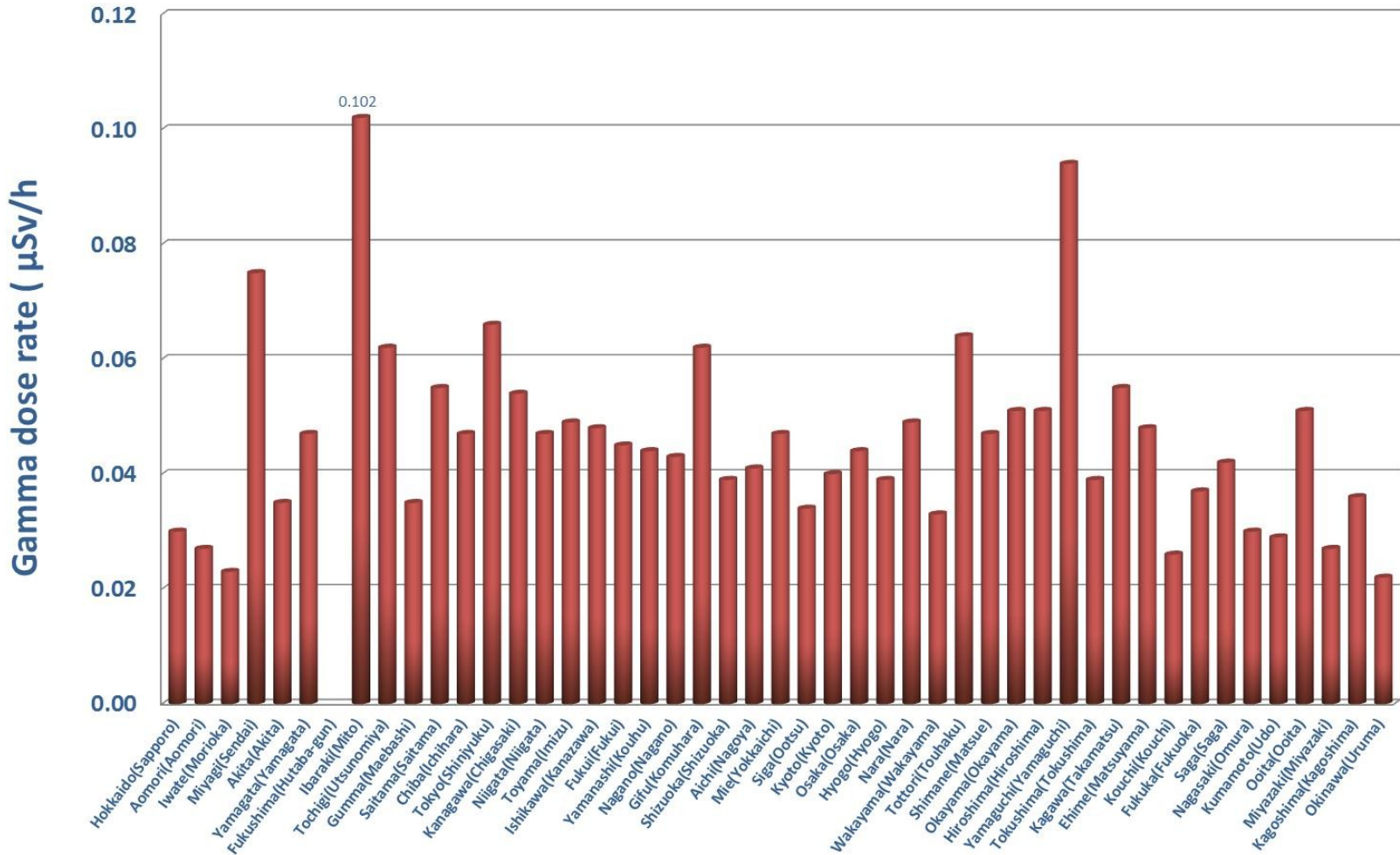
Radiologie

0.1 μSv
Background



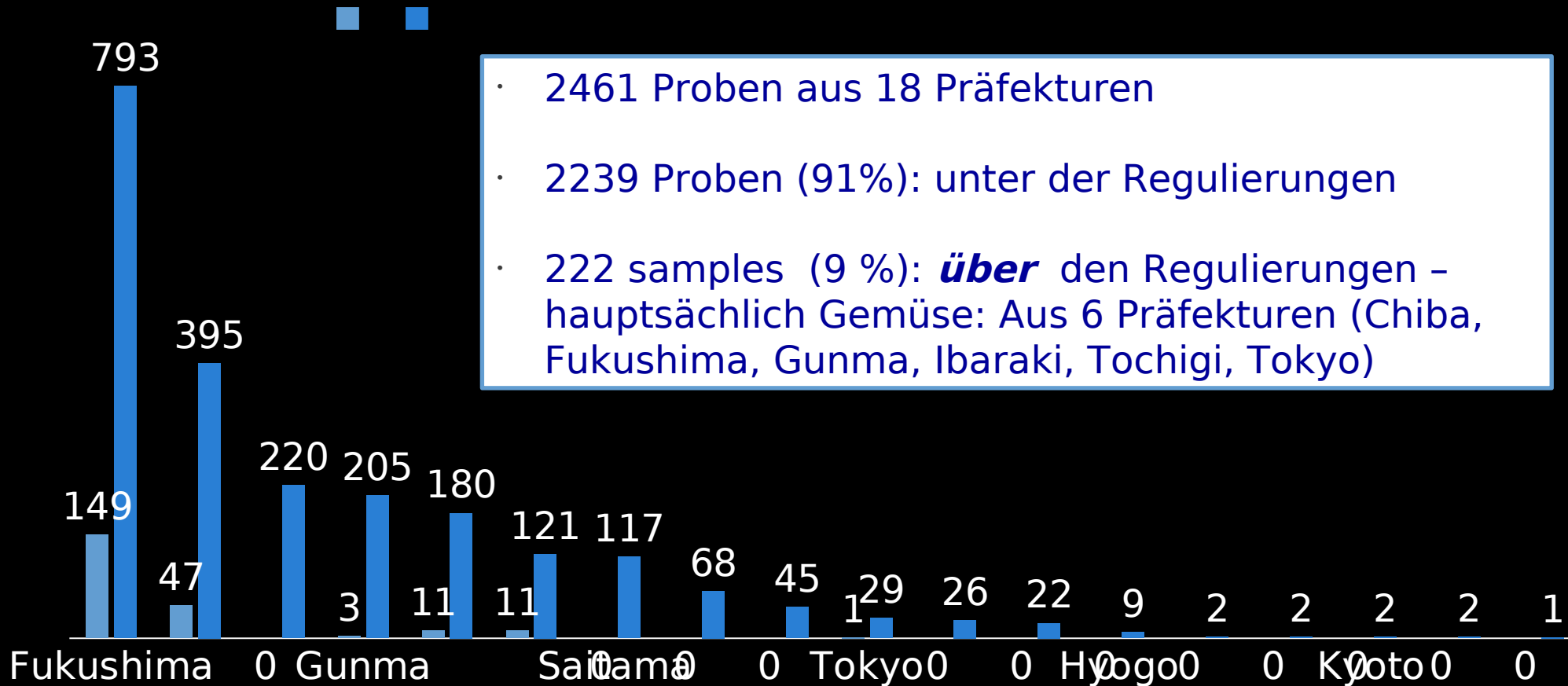
Radiologie

External Gamma Dose Rate (nSv/h) on 04 May 2011 at 08:00 UTC
in the 47 Prefectures - Log scale

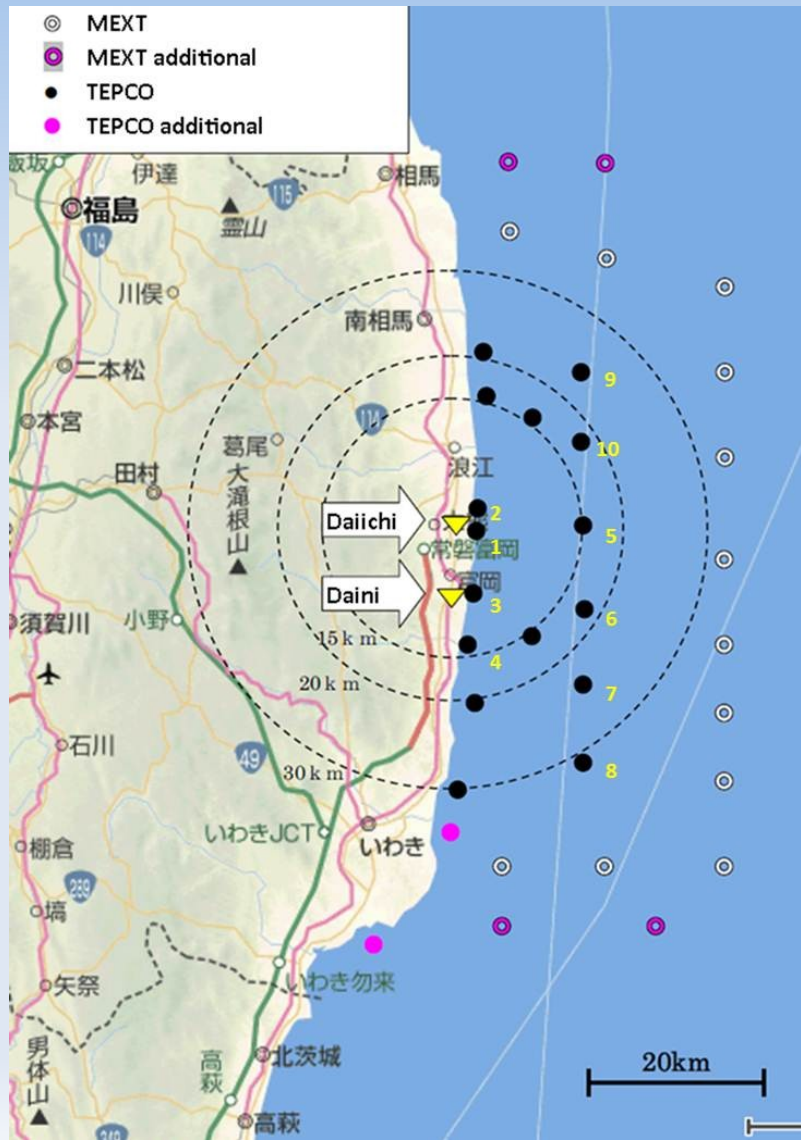


Radiologie

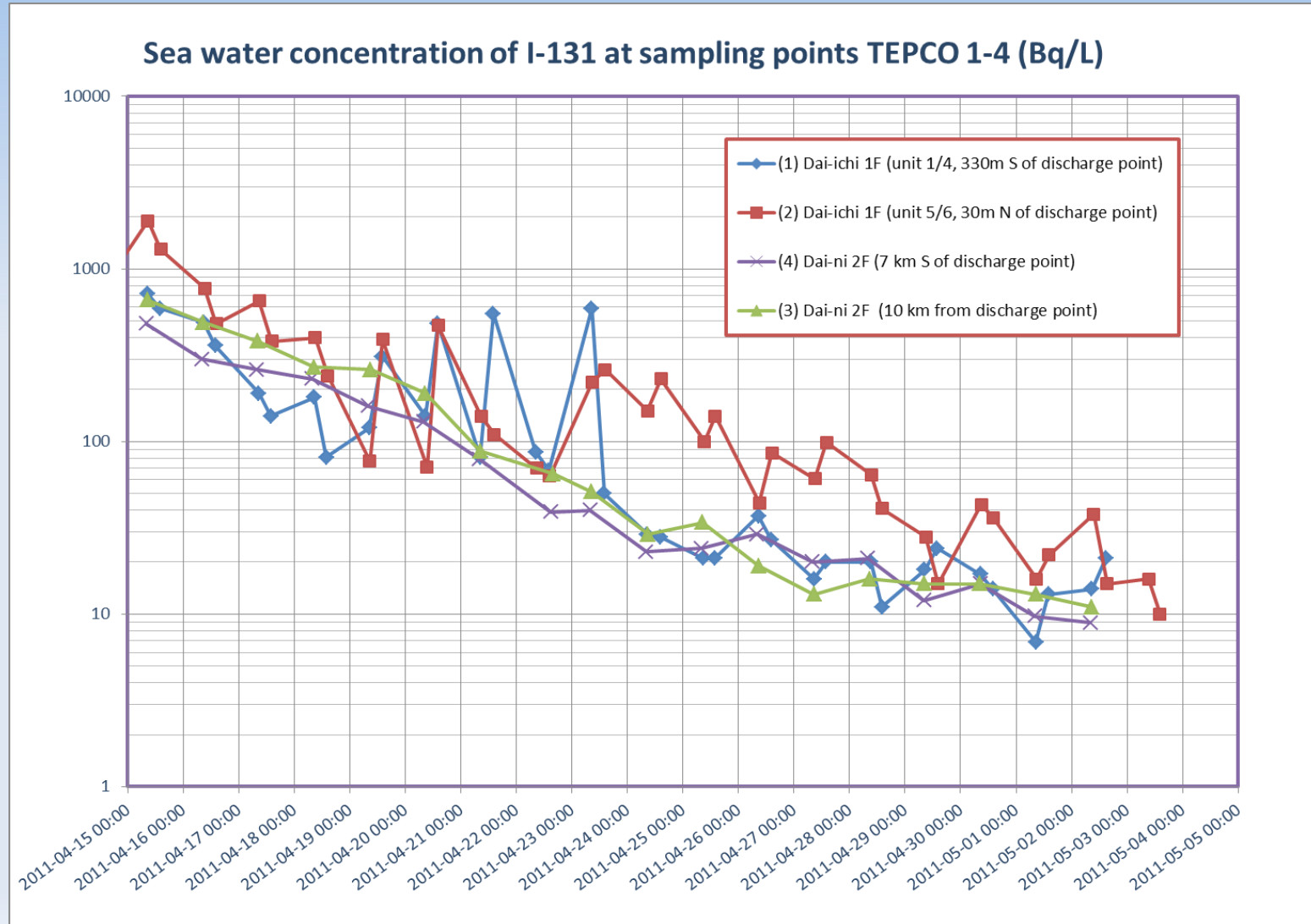
Food Monitoring 19 March to 3 May, MHLW



Radiologie

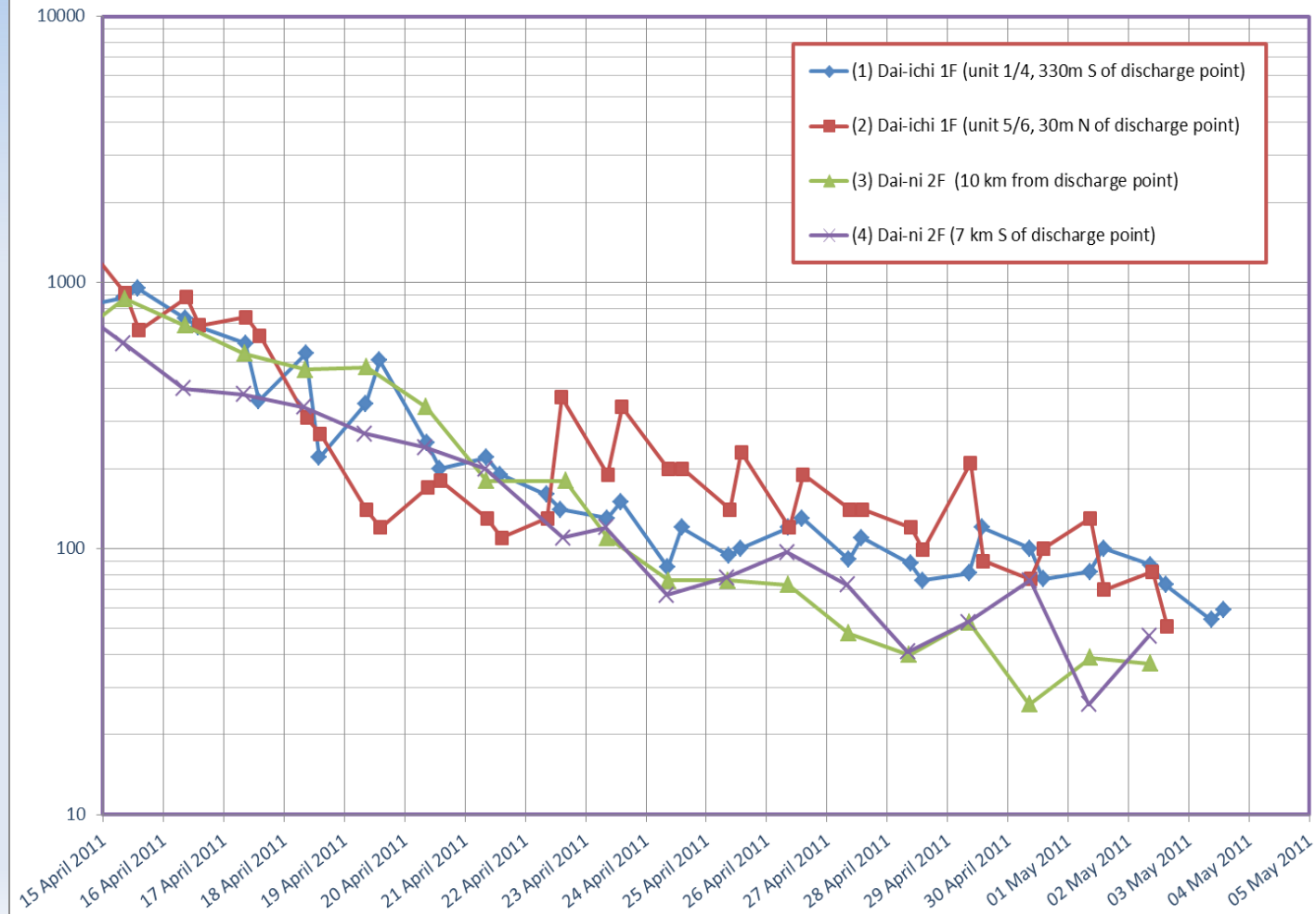


Radiologie



Radiologie

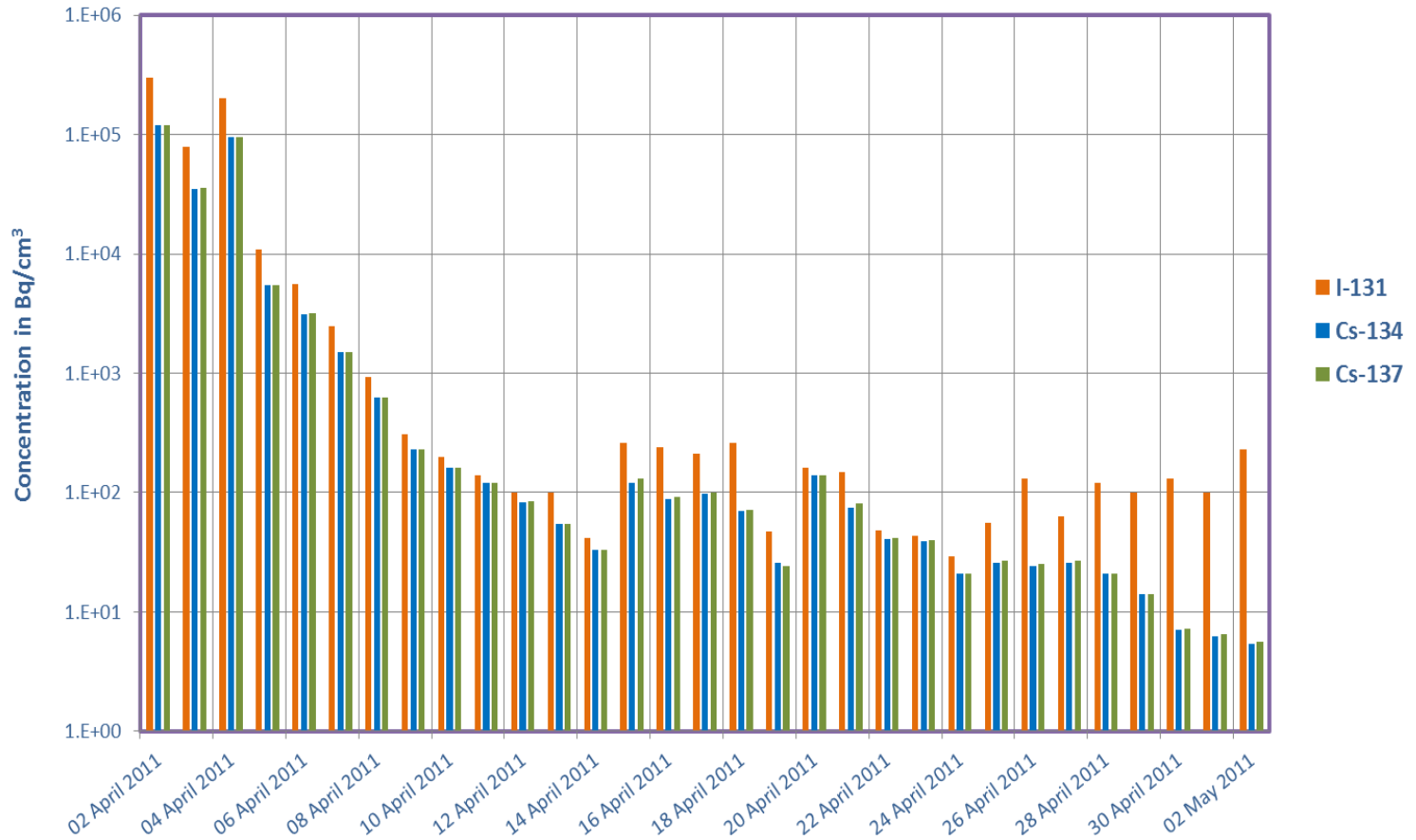
Sea water concentration of Cs-137 at sampling points TEPCO 1-4 (Bq/L)



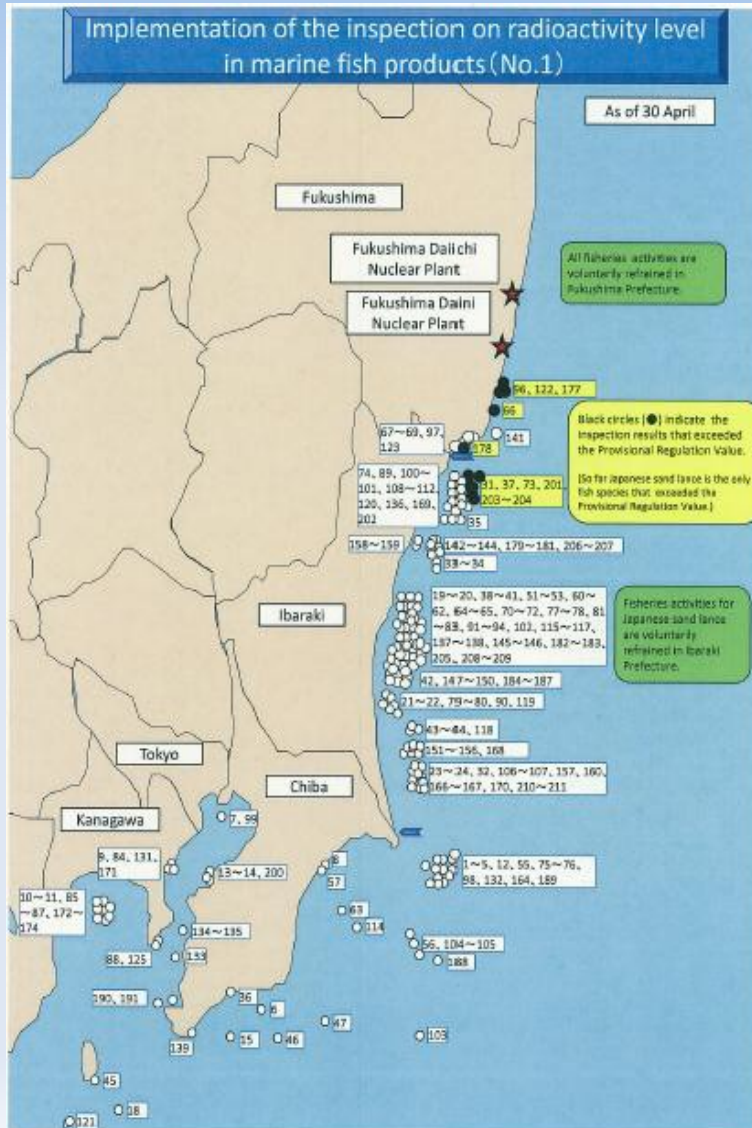
Radiologie

I-131, Cs-137 & Cs-134 (Bq/cm³) concentration of seawater at the screen of Unit 2

The concentrations are in logarithmic scale



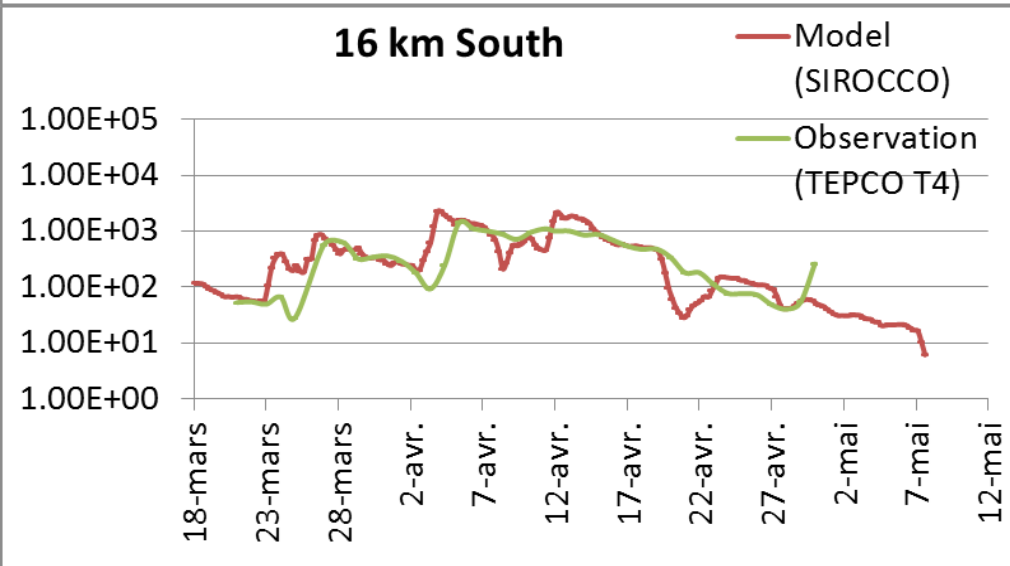
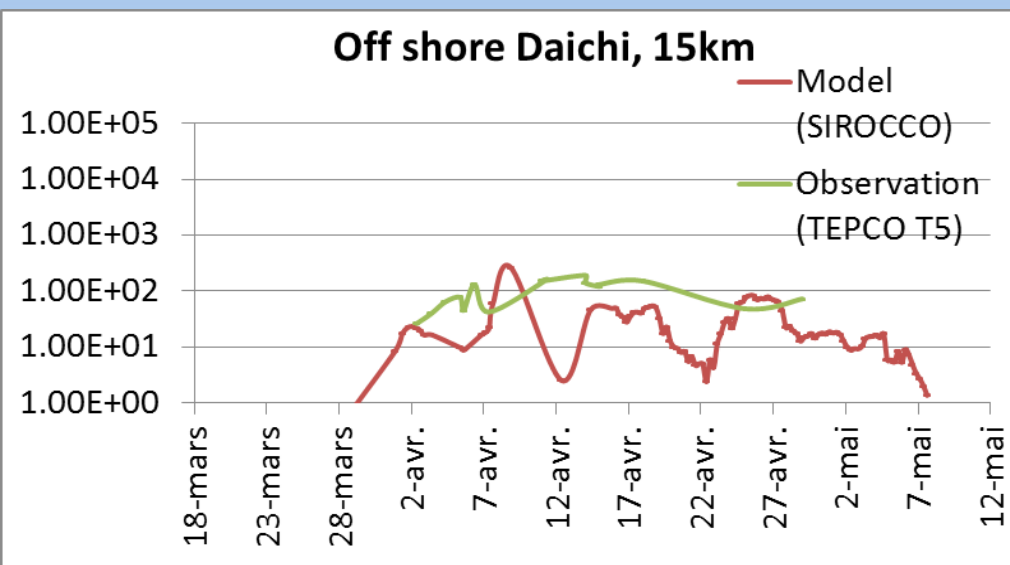
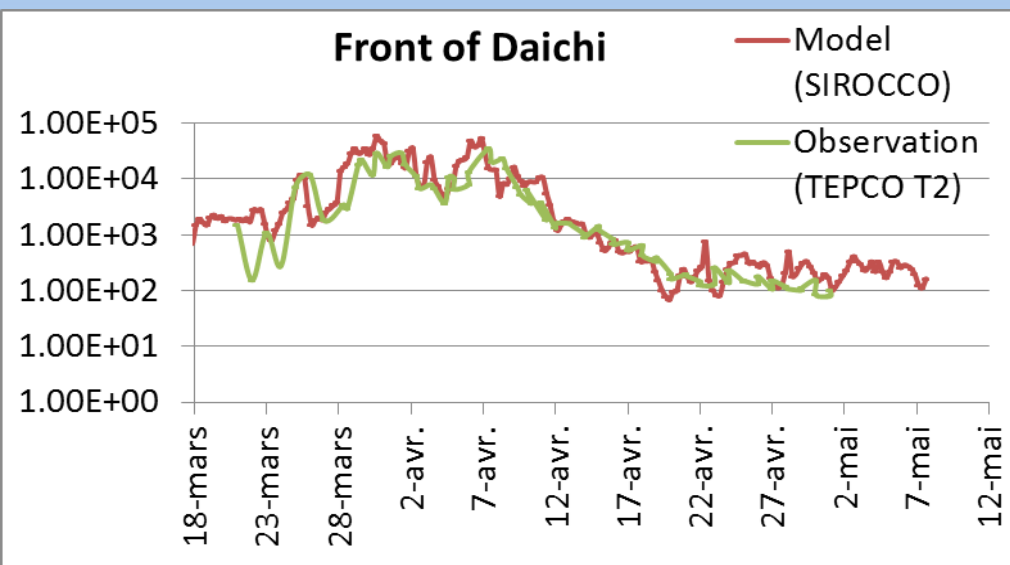
Radiologie



Nur 3 Proben liegen über den festgelegten Werten für Cs-134 und Cs-137.
Nur Sandaale

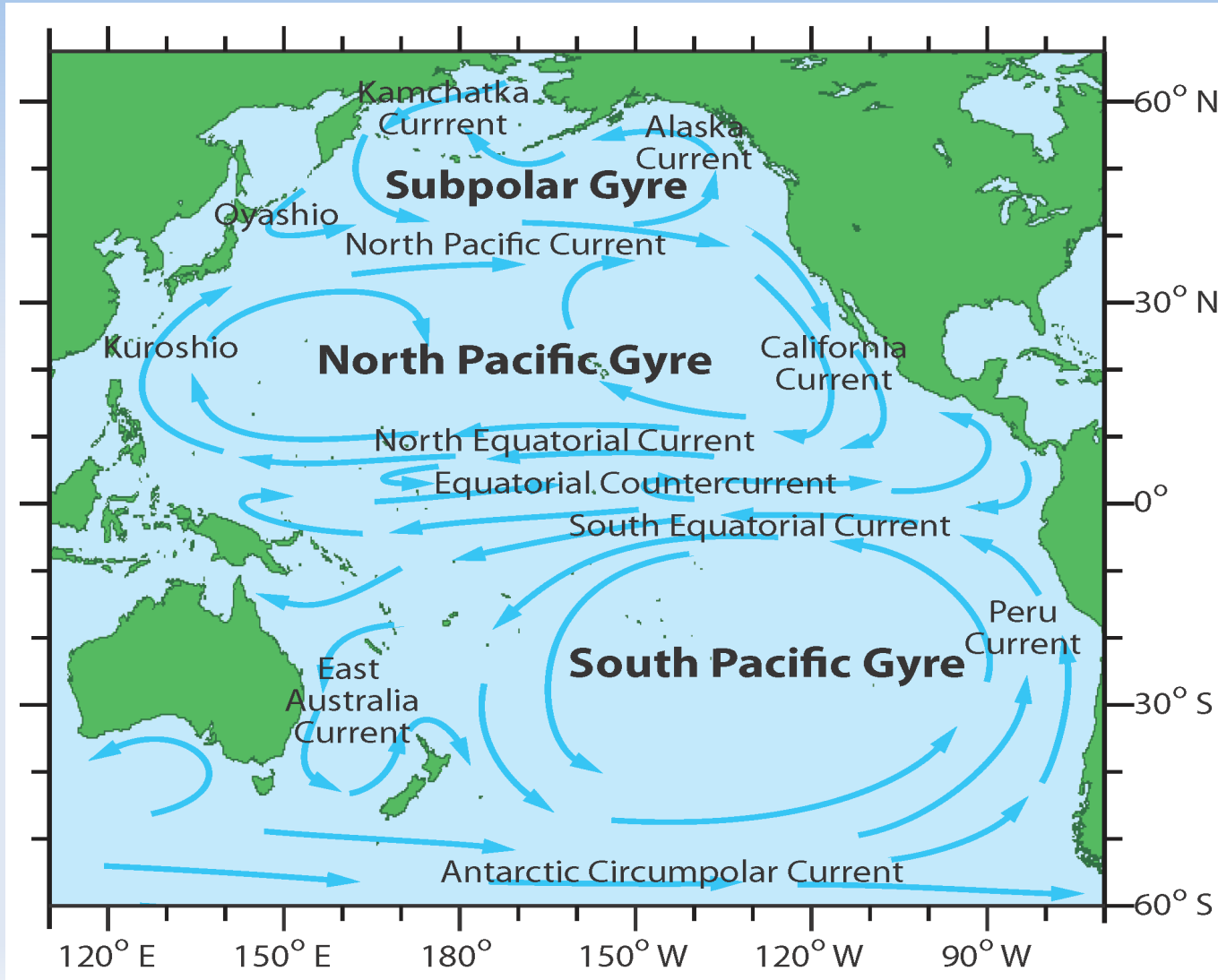


Radiologie



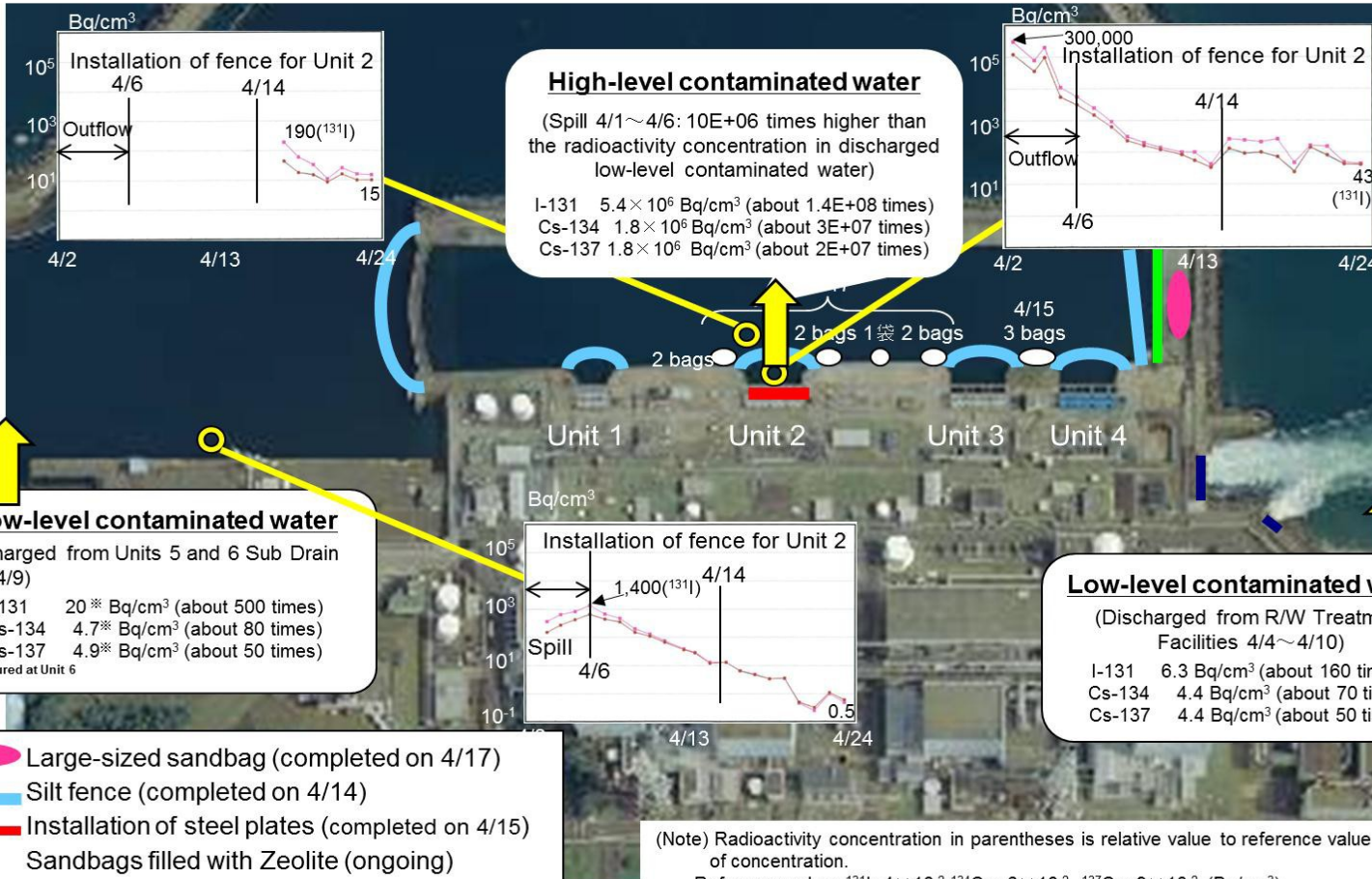
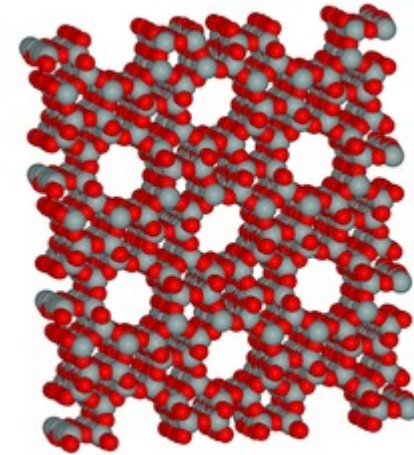
<http://sirocco.omp.obs-mip.fr/outils/Symphonie/>

Radiologie



Maßnahmen

Measures for preventing spread of the liquid including radioactive materials



Low-level contaminated water

(Discharged from Units 5 and 6 Sub Drain 4/4~4/9)

I-131 20* Bq/cm³ (about 500 times)
 Cs-134 4.7* Bq/cm³ (about 80 times)
 Cs-137 4.9* Bq/cm³ (about 50 times)

* measured at Unit 6

Low-level contaminated water

(Discharged from R/W Treatment Facilities 4/4~4/10)

I-131 6.3 Bq/cm³ (about 160 times)
 Cs-134 4.4 Bq/cm³ (about 70 times)
 Cs-137 4.4 Bq/cm³ (about 50 times)

(Note) Radioactivity concentration in parentheses is relative value of concentration.

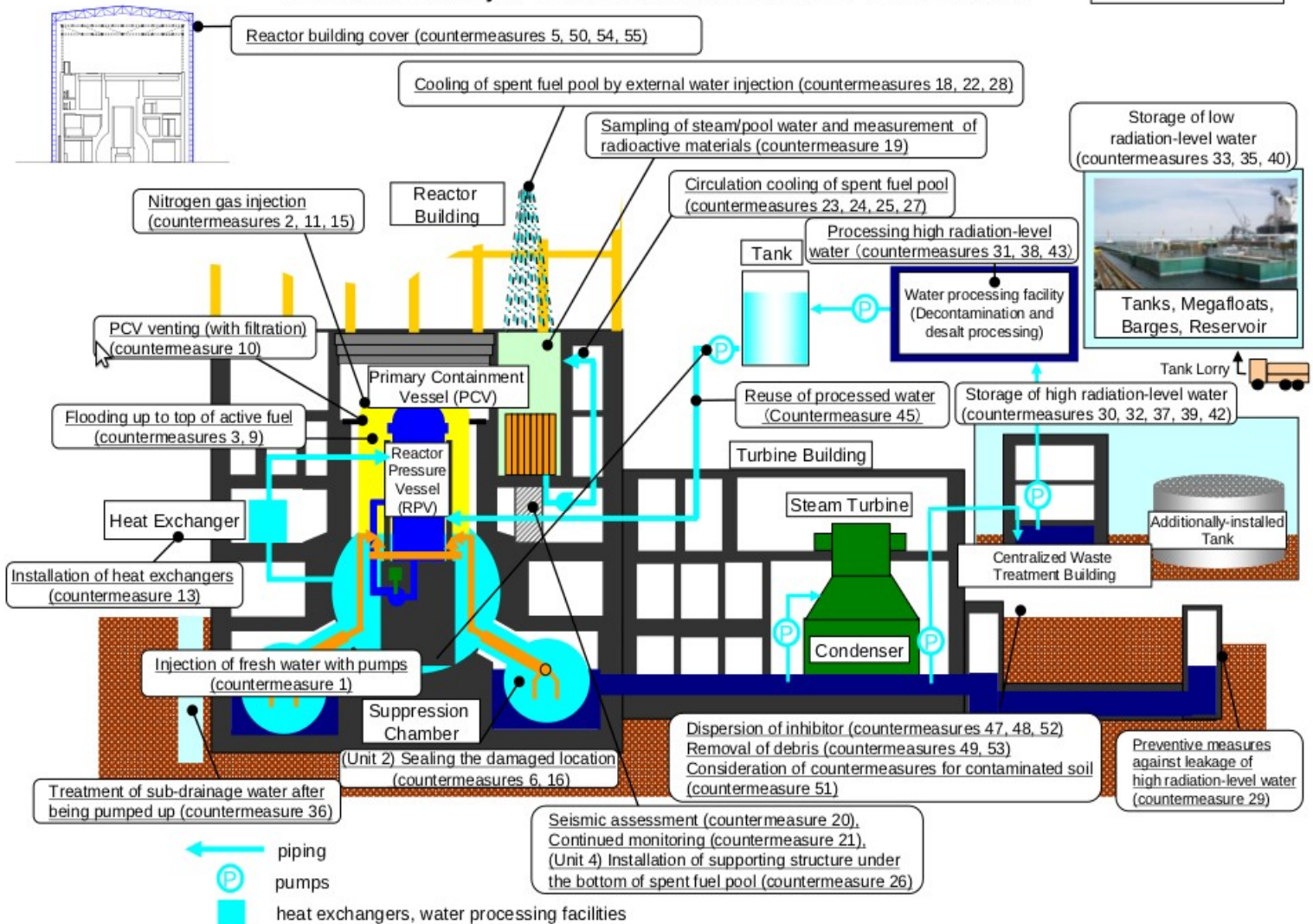
Reference value: ¹³¹I: 4×10^{-2} , ¹³⁴Cs: 6×10^{-2} , ¹³⁷Cs: 9×10^{-2} (Bq/cm³)

(Note) Red lines in graph: ¹³¹I, blue lines: ¹³⁴Cs, brown lines: ¹³⁷Cs

Unit	1	2	3	4
Power (MWe /MWth)	460/1380	784/2381	784/2381	784/2381
Type of Reactor	BWR-3	BWR-4	BWR-4	BWR-4
Status at time of EQ	In service – auto shutdown	In service – auto shutdown	In service – auto shutdown	Outage
Core and fuel integrity	Damaged	Damaged	Damaged	No fuel in the Reactor
RPV & RCS integrity	RPV temperature decreasing	RPV temperature stable	RPV temperature stable	Not applicable due to outage plant status
Containment integrity	No information	Damage suspected	Damage suspected	
AC Power	AC power available - power to instrumentation – Lighting to Central Control Room	AC power available – power to instrumentation – Lighting to Central Control Room	AC power available – power to instrumentation – Lighting to Central Control Room	AC power available – power to instrumentation – Lighting to Central Control Room
Building	Severe damage	Slight damage	Severe damage	Severe damage
Water level of RPV	Around half of Fuel is uncovered	Around half of Fuel is uncovered	Around half of Fuel is uncovered	Not applicable due to outage plant status
Pressure of RPV	Slowly increasing	Stable	Stable	
CV Pressure Drywell	Stable	Stable	Stable	
Water injection to RPV	Injection of freshwater – via mobile electric pump with off-site power	Injection of freshwater – via mobile electric pump with off-site power	Injection of freshwater – via mobile electric pump with off-site power	
Water injection to CV	No information	No information	No information	
Spent Fuel Pool Status	Fresh water injection by concrete pump truck	Freshwater injection to the Fuel Pool Cooling Line	Freshwater injection via Fuel Pool Cooling Line and Periodic spraying	Fresh water injection by concrete pump truck

Overview of Major Countermeasures in the Power Station

Reference 2



Quellen

- IAEA
- JAIF
- VBG
- AREVA
- ATI
- TEPCO
- SIROCCO